18 Septembre 2018

## IAEA SAFETY STANDARDS

for protecting people and the environment

Step 7

First review of the draft publication by the review committees

**Reviewed in NSOC (Shaw)** 

# Seismic Hazards in Site Evaluation for Nuclear Installations

## **DRAFT SAFETY GUIDE No. DS 507**

Revision of Safety Guide SSG-9

### Contents

Со	ntents	1
1.	INTRODUCTION	1
	BACKGROUND	1
	OBJECTIVE	1
	SCOPE	2
	STRUCTURE	3
2.	GENERAL RECOMMENDATIONS	5
3.	DATABASE OF INFORMATION AND INVESTIGATIONS	9
	GENERAL	9
	GEOLOGICAL, GEOPHYSICAL AND GEOTECHNICAL DATABASE	12
	Regional investigations	12
	Near regional investigations	13
	Site vicinity investigations	14
	Site area investigations	15
	SEISMOLOGICAL DATABASE	17
	Pre-historical and pre-instrumental historical earthquake data	17
	Instrumental historical earthquake data	18
	Project Earthquake Catalogue	19
	Site specific instrumental data	20
4.	CONSTRUCTION OF SEISMIC SOURCE MODELS	22
	GENERAL	. 22
	SEISMOGENIC STRUCTURES (IDENTIFIED SEISMIC SOURCES)	23
	Identification	23
	Characterization	. 24
	ZONES OF DIFFUSE SEISMICITY	25
	Identification	25
	Characterization	26
5.	VIBRATORY GROUND MOTION ESTIMATION METHODS	. 28
	GENERAL	. 28
	GROUND MOTION PREDICTION EQUATIONS	. 29
	Selection criteria	29

	ANTS	
9.	EVALUATION OF SEISMIC HAZARDS FOR NUCLEAR INSTALLATIONS OTHER THAN NUCLEAR POWE	
	Volcano related phenomena	
	Failure of water retaining structures (dam break)	
	Collapse due to cavities and subsidence phenomena	
	Slope stability	
	Liquefaction potential	
C	Tsunamis	
	EVALUATION OF OTHER HAZARDS ASSOCIATED WITH EARTHQUAKES	
	AULT DISPLACEMENT	
	Ground motion for base isolated structures, buried structures and fuel pools	
	Vertical ground motion	
	Ground motion duration	
	Time histories	
	Spectral representations	
	Site response analysis	
2	Parameters and control point	
	EISMIC VIBRATORY GROUND MOTION HAZARDS	
	HER ASSOCIATED SEISMIC HAZARDS	
8. ОТ	PARAMETERS FROM THE VIBRATORY GROUND MOTION ANALYSIS, FAULT DISPLACEMENT AND	16
(	CAPABLE FAULT ISSUES FOR SITES WITH EXISTING NUCLEAR INSTALLATIONS	. 44
	CAPABLE FAULT ISSUES FOR PROPOSED NEW SITES	
	Investigations necessary to determine capability	
	Definition	. 42
(	CAPABLE FAULTS	
(	GENERAL	. 41
7.	EVALUATION OF THE POTENTIAL FOR FAULT DISPLACEMENT AT THE SITE	. 41
S	SITE RESPONSE ANALYSIS	
	DETERMINISTIC SEISMIC HAZARD ASSESSMENT	
F	PROBABILISTIC SEISMIC HAZARD ASSESSMENT	. 33
C	GENERAL	. 33
6.	VIBRATORY GROUND MOTION SEISMIC HAZARD ANALYSIS	. 33
(	GROUND MOTION SIMULATION METHODS	. 31
	Epistemic uncertainties of the technically defensible interpretations	. 31

GENERAL	. 52	
SCREENING PROCESS	. 52	
CATEGORIZATION PROCESS	. 53	
VIBRATORY GROUND MOTION AND OTHERS HAZARD ANALYSIS	. 54	
Vibratory ground motion hazard assessment	. 54	
Associated geological and geotechnical seismic hazards	. 55	
10. PROJECT MANAGEMENT SYSTEM	. 56	
GENERAL ASPECTS OF PROJECT ORGANIZATION	. 56	
SPECIFIC ASPECTS OF PROJECT ORGANIZATION	. 57	
ENGINEERING USES AND OUTPUT SPECIFICATION	. 58	
INDEPENDENT PEER REVIEW	. 60	
REFERENCES		
ANNEX-TYPICAL OUTPUT OF PROBABILISTIC SEISMIC HAZARD ANALYSES		
TABLE A–1. TYPICAL OUTPUT OF PROBABILISTIC SEISMIC HAZARD ANALYSES		
DEFINITIONS		
CONTRIBUTORS TO DRAFTING AND REVIEW		

#### **1. INTRODUCTION**

#### BACKGROUND

1.1. This Safety Guide provide recommendations on how to meet the requirements of IAEA Safety Standards Series No. SSR-1, Site Evaluation for Nuclear Installations [1] in relation to the procedures for the evaluation of hazards generated by earthquakes affecting nuclear power plants and other nuclear installations. This publication is a revision of IAEA Safety Standards Series No. SSG-9, Seismic Hazards in Site Evaluation for Nuclear Installations<sup>1</sup>, which it supersedes.

1.2. The previous version of the IAEA Safety Guide on the evaluation of the seismic hazards was extensively used by Member States and positive feedback of their application was received from the IAEA reviews of the seismic safety of nuclear installations worldwide.

1.3. This Safety Guide incorporates:

- (a) Progress in relation to practice and research in the evaluation of seismic hazards, as well as the regulatory practice in Member States, considering the lessons learned from the occurrence of recent strong earthquakes that affected nuclear installations;
- (b) Recent developments and regulatory requirements on risk informed and performance based approaches for assessing the safety of nuclear installations;
- (c) Experience and results from seismic hazard evaluations performed for the evaluation of new and existing sites for nuclear installations in Member States;
- (d) A more coordinated treatment of the seismically induced associated geological and geotechnical hazards and concomitant events;
- (e) A more consistent approach for considering the diversity of professional judgment by experts and the treatment of the uncertainties involved in the process of evaluating seismic hazards.

1.4. This revision also provides a clearer separation between the process for assessing the seismic hazards at a specific site, and the process for defining the related basis for design and evaluation of the nuclear installation. Thus, it bridges gaps and avoids undue overlapping on recommendations related to the two processes which correspond to and are performed at different stages of the lifetime of the nuclear installation.

#### OBJECTIVE

1.5. The objective of this Safety Guide is to provide recommendations on how to meet the

<sup>&</sup>lt;sup>1</sup> INTERNATIONAL ATOMIC ENERGY AGENCY, Seismic Hazards in Site Evaluation for Nuclear Installations, IAEA Safety Standards Series No. SSG-9, IAEA, Vienna (2010)

requirements established in SSR-1 [1] in relation to the evaluation of hazards generated by earthquakes that might affect a nuclear installation site and, in particular, on how to determine:

- (a) The vibratory ground motion hazards necessary to establish the design basis ground motions and other relevant parameters for the design and safety assessment of both new and existing nuclear installations;
- (b) The potential for, and rate of, fault displacement phenomena that could affect the feasibility of the site for a new nuclear installation or the safe operation of the existing installation at that site;
- (c) The earthquake parameters necessary for assessing the associated geological and geotechnical hazards (e.g. soil liquefaction, landslides and differential settlements, and collapse due to cavities and subsidence phenomena) and concomitants events (e.g. external flooding phenomena, as tsunami, and fires).

1.6. This Safety Guide is intended for use by regulatory bodies responsible for establishing regulatory requirements, and for operating organizations directly responsible for the evaluation of seismic hazards at a nuclear installation site.

#### SCOPE

1.7. The recommendations in this Safety Guide are intended to be used for the evaluation of seismic hazards for nuclear installations in any seismotectonic environment.

1.8. This Safety Guide addresses the following nuclear installations [2]:

- Land based stationary nuclear power plants;
- Research reactors (including subcritical and critical assemblies) and any adjoining radioisotope production facilities;
- Storage facilities for spent fuel;
- Facilities for the enrichment of uranium;
- Nuclear fuel fabrication facilities;
- Conversion facilities;
- Facilities for the reprocessing of spent fuel;

The methodologies recommended for nuclear power plants are applicable to other nuclear installations by means of a graded approach, whereby these recommendations can be customized to suit the needs of nuclear installations of different types in accordance with the potential radiological consequences of their failure when subjected to seismic loads. The recommended approach is to start with attributes relating to nuclear power plants and to modify the application of the recommendations until they are commensurate with installations with which lesser radiological consequences are associated<sup>2</sup>. If no grading is performed, the

 $<sup>^{2}</sup>$  For sites at which nuclear installations of different types are collocated, particular consideration should be given to using a graded approach.

recommendations relating to nuclear power plants should be applied to other types of nuclear installations. Also, the level of detail, the efforts and resources applied for evaluating the seismic hazards at an existing installation site should be commensurate with the time remaining until the planned decommissioning of the nuclear installation and the defined and established risk and performance goals.

1.9. For the purpose of this Safety Guide, existing nuclear installations are those installations that are: (a) at the operational stage (including long term operation and extended temporary shutdown periods); (b) at a pre-operational stage for which the construction of structures, the manufacturing, installation and/or assembly of components and systems, and commissioning activities are significantly advanced or fully completed; or (c) at temporary or permanent shutdown stage while radioactive material is still within the facility (in the core or the pool).

1.10. Earthquakes generate several direct and indirect phenomena. From vibratory ground motions to associated geological and geotechnical hazards, such as permanent ground displacement (e.g. soil liquefaction, slope instability, tectonic and non-tectonic subsidence, cavities leading to ground collapse, and settlements) to concomitant events such as seismically induced fires and floods. This Safety Guide provides guidance on how to consistently characterize and define the related seismic parameters that are necessary for evaluating the associated geological and geotechnical hazards and concomitant events as described in IAEA Safety Standards Series No. NS-G-3.6, Geotechnical Aspects of Site Evaluation and Foundations for Nuclear Power Plants [3], and IAEA Safety Standards Series No. SSG-18, Meteorological and Hydrological Hazards in Site Evaluation for Nuclear Installations [4].

1.11. This Safety Guide addresses aspects related to the evaluation of hazards generated by earthquakes that might affect the site. This evaluation will be performed during the site selection and/or site evaluation stages, possibly prior to the availability of information related to the design characteristics of the nuclear installation, or during the operation stage of an existing nuclear installation. Thus, the seismic hazards may need to be determined independently of the characteristics of the nuclear installation that is to be installed. Recommendations for the determination of the related basis for the design and evaluation of the nuclear installation(s) through the use and application of appropriate criteria are provided in IAEA Safety Standards Series No. NS-G-1.6, Seismic Design and Qualification for Nuclear Power Plants [5].

#### STRUCTURE

1.12. Recommendations of a general nature are provided in Section 2. Recommendations on the acquisition of a database containing the information needed to evaluate and address all hazards associated with earthquakes are provided in Section 3. Section 4 covers the use of this database for the construction of the seismic source models specific to the site of the nuclear installation. Section 5 reviews available methods for conducting vibratory ground motion

analysis. Sections 6 provides recommendations on probabilistic and deterministic methods of evaluating vibratory ground motion hazards. Section 7 presents methods for evaluation of the potential for fault displacement. Section 8 provides recommendations on the development of design basis ground motion and fault displacement.

1.13. Sections 3 to 8 focus primarily on nuclear power plants. Section 9 provides recommendations on the evaluation of seismic hazards for nuclear installations other than nuclear power plants using a graded approach. Section 10 addresses project management, including quality assurance and peer review requirements. The Annex provides an example of typical output deriving from probabilistic seismic hazard analyses.

#### 2. GENERAL RECOMMENDATIONS

2.1. SSR-1 [1] establishes the following requirements:

**Requirement 2: Site safety objectives in site evaluation for nuclear installations** 

"The safety objectives in site evaluation for nuclear installations shall be (a) to characterize the natural and human induced hazards that might challenge the safety of the nuclear installation and (b) to provide adequate input for demonstration of the sufficient protection of people and the environment from harmful effects of ionizing radiation."

#### **Requirement 15: Evaluation of fault capability**

"Faults beyond a certain size and within a certain distance of the nuclear installation that are significant to safety shall be evaluated to identify whether these faults are to be considered capable faults. For capable faults, potential challenges to the safety of the nuclear installation in terms of ground motion and/or fault displacement hazard shall be evaluated."...

**Requirement 16: Evaluation of ground motion hazards** 

"An evaluation of ground motion hazards shall be conducted to provide the input needed for the seismic design or safety upgrading of the structures, systems and components of the nuclear installation, as well as for performing the deterministic and/or probabilistic safety analyses necessary during the lifetime of the nuclear installation."...

In accordance with these requirements and in line with recognized international practice, the geological, geophysical and seismological characteristics of the geographical region around the site and the geotechnical characteristics of the site area should be investigated for evaluating the seismic hazards at the nuclear installation site.

2.2. The size of the region to be analyzed should be determined based on the types, magnitudes and distances from the source to the site of potentially hazardous phenomena generated by earthquakes that might have an impact on the safety of the nuclear installation. Thus, the region should be of sufficient extent to include all seismic sources that could reasonably be expected to contribute to the seismic hazards at the site. It does not necessarily have predetermined, uniform dimensions and it should be defined on the basis of the specific site and region conditions. If necessary, the region should include areas extending beyond national borders as well as relevant offshore area.

2.3. The size of the region to be investigated, the type of information and data to be collected and the scope and detail of the investigations to be performed should be defined at the beginning of the project of seismic hazard evaluation. The acquired database should be sufficient for characterizing, from a seismotectonic point of view, relevant features to the

seismic hazard evaluation that are located in other States or in offshore areas.

2.4. The evaluation of the seismic hazards for a nuclear installation site should be done through implementation of a specific project plan for which clear and detailed objectives are defined, and with a project management organization and structure to provide for coherency and consistency in the database and a reasonable basis on which to compare results for all types of seismic hazards. This project plan should include an independent peer review process. It should be carried out by a multidisciplinary team of experts, including geologists, seismologists, geophysicists, seismic hazard specialists, engineers and possibly other experts (e.g. historians) as necessary. The members of the team for the seismic hazard evaluation project and its independent peer review should demonstrate the expertise and experience commensurate with their role in the project. Figure 1 shows the seismic hazard evaluation process as a whole and the general steps and sequence to be followed.

2.5. The general approach to seismic hazard evaluation should be directed towards a realistic identification, quantification, treatment and reduction of the uncertainties at various stages of the project. Experience shows that the most effective way of achieving this is to collect sufficient reliable and relevant site specific data. There is generally a compromise between the time and effort necessary to compile a detailed, reliable and relevant database and the degree of uncertainty that the analyst should take into consideration at each step of the process. Thus, a lower level of effort for database development in relation to seismic source and ground motion characterization will result in increased uncertainty in the final obtained results.

2.6. Therefore, an adequate method for identification, quantification and treatment of the uncertainties should be formulated at the beginning of the project. In general, significant uncertainties are involved in the seismic hazard assessment process. Basically, two types of uncertainties are identified for practical application on seismic hazard evaluation: (i) the aleatory variability of the seismic process, that is intrinsic or random and irreducible through collecting more data because it represents an inherent natural feature, and (ii) the epistemic uncertainty, that is extrinsic in nature and is associated with modelling or lack of knowledge and can be reduced through the acquisition of additional data, further research and interaction between experts considering the diversity of their professional judgment.

2.7. Site specific, sufficient and reliable data should be collected in the seismic hazard evaluation process. However, part of the data used indirectly in seismic hazard analysis might not be site specific, in particular the strong motion data used to develop the ground motion prediction equations (GMPEs). Therefore, it should be recognized that part of the uncertainty is irreducible with respect to performing additional site specific investigations.

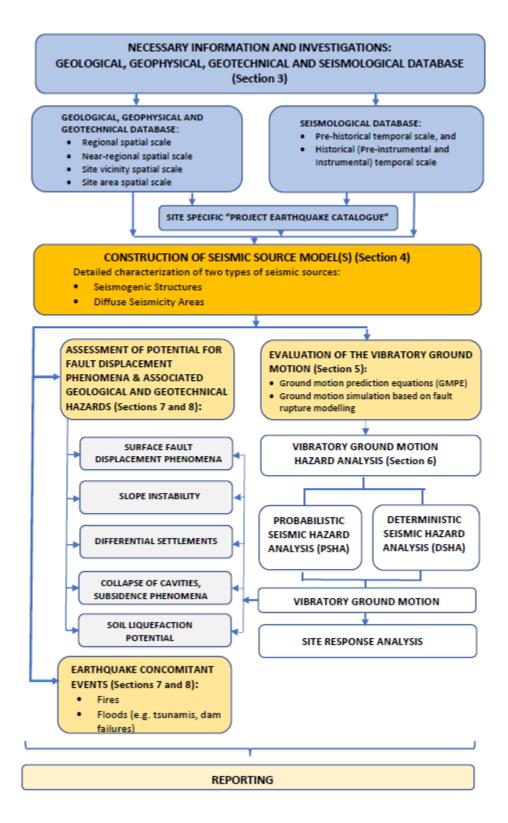


FIG 1: Flow chart for the seismic hazard evaluation process for nuclear installations

2.8. It is recognized that one of the many sources of epistemic uncertainties in seismic hazard evaluation is the differences in interpretation of the available data and the diversity of professional judgment of the experts participating in the hazard assessment process. Care should be taken to avoid bias in these interpretations. Expert judgment should not be used as a substitute for acquiring new data. The project team for the seismic hazard evaluation should not promote any one expert hypothesis or model. It should, however, evaluate all viable hypotheses and models using the data compiled. Then, an integrated evaluation that incorporates both the existing knowledge and the uncertainties of the respecting data should be developed.

2.9. In order to address the diversity of scientific interpretations, it is recommended that the centre, body and range of the technically defensible interpretations [6] are properly captured. For this purpose, multidisciplinary teams of experts with appropriate qualifications in each of the relevant areas should be involved to develop a model that robustly represents the epistemic uncertainties related to methods and models employed in the seismic hazard evaluation. Approaches that use expert elicitation should avoid putting these experts in a role that might jeopardize the significance of their professional judgements as supported by the available earth science data. Also, the adequate consideration of uncertainties using appropriate (e.g., conservative or best estimate) and credible models, methods and scenarios, based on the technically defensible interpretations concept should be made given the evaluation framework (i.e. deterministic or probabilistic) and the target confidence levels.

2.10. A set of quality management documents should be prepared and properly updated during the seismic hazard evaluation process. While technical references that explain these processes are very useful, the guidance they provide might be interpreted in different ways. An unambiguous set of project specific quality documents (quality plan, work plan and procedures) should be prepared that contain all the criteria that are applicable in the project as well as the documentation recording all expert interpretations. More detailed guidance on this topic is provided in Section 10.

2.11. As indicated in para. 2.8, uncertainties that cannot be reduced by means of specific site investigations (e.g. uncertainties arising from the use of GMPEs derived for other parts of the world) do not permit hazard values to decrease below certain threshold values. For this reason, and regardless of any lower apparent exposure to seismic hazard, a minimum vibratory ground motion level should be recognized as the lower limit to be used for seismic design, safety assessment and/or seismic safety evaluation of any nuclear installation and that minimum level should be adopted when applying the recommendations in NS-G-1.6 [5].

#### 3. DATABASE OF INFORMATION AND INVESTIGATIONS

#### GENERAL

3.1. A comprehensive and integrated database of geological, geophysical, geotechnical and seismological information should be compiled in a coherent form for evaluating and resolving issues relating to all hazards generated by earthquakes.

3.2. It should be ensured that each element of each individual database has been investigated as fully as possible before an integration of the various elements into a unique consolidated database is attempted. The integrated database should include all relevant information; not only geological, geophysical, geotechnical and seismological data, but also any other information that is relevant to evaluating the vibratory ground motion, the fault displacement phenomena, the associated geological and geotechnical hazards and the concomitant events affecting the site.

3.3. The data and information to be acquired for the geological, geophysical, geotechnical and seismological database should cover a geographical region and a temporal scale commensurate with the potential of the seismic hazards to affect the safety of the nuclear installation at the site.

In relation to the geographical area of interest to be investigated, SSR-1 [1] states that:

- "The site and the region shall be investigated with regard to the characteristics that can impact safety of nuclear installation and the potential radiological impact of the nuclear installation on people and the environment." (Requirement 5 of SSR-1 [1])
- "Natural phenomena as well as human activities in the region with potential to induce hazards at the site that might affect the safety of the nuclear installation shall be identified and evaluated. The extent of this evaluation shall be graded in accordance with the safety significance of the potential hazards at the site." (para. 4.14 of SSR-1 [1])
- "Characteristics of the natural environment in the region that can be affected by the potential radiological impact of the nuclear installation shall be investigated and assessed, for all operational states and accident conditions and for all stages of the lifetime of the nuclear installation (See Section 6)." (para. 4.15 of SSR-1 [1])
- "The size of the region to be investigated shall be defined for each of the external natural hazards and associated activities. Both the magnitude of the hazard and the distance from the source of the hazard to the site shall be considered in determining the size of the region to be investigated. For certain external natural phenomena, such as tsunamis and volcanic phenomena, it shall be ensured that the size of the region surrounding the site is sufficiently large to address the potential effects of these hazards at the site." (para. 4.16 of SSR-1 [1])

- "The site and the region shall be studied to evaluate the present and foreseeable future characteristics that could have an impact on installation safety of nuclear installation. This includes potential changes in the magnitude and/or frequency of the natural hazards, the distribution of the population in the region, the present and future use of land and water, the further development of existing nuclear installations or the construction of facilities that can impact on the safety of the nuclear installation or the feasibility of planning the implementation of emergency response actions effectively." (para. 4.17 of SSR-1 [1])

In relation to the temporal scale of the investigations, SSR-1 [1] states that:

- "The data necessary to perform an assessment of external natural and human induced hazards and to assess both the impact of the environment on the nuclear installation safety and the impact of the nuclear installation on public and the environment shall be collected." (Requirement 14 of SSR-1 [1])
- "Prehistoric, historical and instrumentally recorded information and records of the occurrences and severity of important natural phenomena shall be obtained, if available, as appropriate for the hazard to be evaluated and shall be analysed for reliability, accuracy, temporal pertinence and completeness." (para. 4.48 of SSR-1 [1])

3.4. The size of the geographical area at the regional scale in which the geological, geophysical, geotechnical and seismological database should be compiled may vary depending on the geological and tectonic setting, and the recommendations provided in para. 2.3 should be used for defining the appropriate size of the region to be investigated.

3.5. The geological, geophysical and geotechnical investigations for evaluating the seismic hazards at the site, should be conducted on four spatial geographical scales — regional, near regional, site vicinity and site area — leading to progressively more detailed investigations, data and information when approaching closer to the nuclear installation site. The detail and type of these data are determined by the different spatial geographical scales. The first three scales of investigation lead primarily to progressively more detailed geological and geophysical data and information. The site area investigations are mainly aimed at developing the geotechnical database.

3.6. Finally, with the completion of the geological, geophysical and geotechnical investigations at the four spatial scales, all potential seismogenic features that have been identified and characterized should be documented in a systematic way to ensure consistency and completeness so that similar attributes for all seismic sources are compiled in the 'Project Fault Catalogue' (or 'Project Fault Portfolio'), including assessment of uncertainties for the fault parameters.

3.7. The seismological database should include all available information and data on earthquake events that have occurred in the region, defined as recommended in para. 3.4, and

they should cover the pre-historical and historical temporal scales. The historical temporal scale should be further subdivided into pre-instrumental and instrumental periods.

3.8. In offshore regions and other areas for which seismological data is poor, adequate investigations should be conducted to fully analyse the tectonic characteristics of the region and to compensate for any lack of or deficiency in the seismological data.

3.9. In the case of investigations for evaluating the potential for earthquake generated tsunamis (see NS-G-3.6 [3]), the geological and seismological investigations should also include the study of seismic sources located at very great distances from the site. Thus, it should be noted that the sources of earthquakes that can generate relevant seismic and tsunami hazards at the site might not be the same. For tsunamis related to earthquake induced submarine landslides, the models used for calculating the ground motion inducing the landslide should be consistent with those used for the seismic hazard evaluation for the nuclear installation for this specific hazard.

3.10. New techniques that have recently emerged for the acquisition and processing of data (e.g. remote sensing, age dating) for identifying and characterizing seismic sources should be implemented. It is also possible that new types of data may be generated as a result of these technological developments. While it is recommended that state-of-the-art, new, updated and recognized technological developments are implemented, such developments should first be checked regarding their adequacy and effectiveness to be used in a nuclear installation site evaluation project.

3.11. Considering that earthquakes produce observable effects on the environment, palaeoseismological studies should be performed, as necessary at any of the four spatial scales, to:

- (a) Identify the seismogenic structures based on the recognition of effects of past earthquakes in the region.
- (b) Improve the completeness of earthquake catalogues for large events, using identification and age dating of fossil earthquakes. For example, observations of trenching across the identified capable faults may be useful in estimating the amount of displacement (e.g. from the thickness of colluvial wedges) and its rate of occurrence (e.g. by using age dating of the sediments). Also, studies of palaeo-liquefaction and palaeo-tsunami can provide evidence of the recurrence and intensity of earthquakes.
- (c) Estimate the maximum potential magnitude (and the associated uncertainty) of a given seismogenic structure, typically based on the maximal dimensions of the structure and displacement per event (estimated by trenching) as well as of the cumulative effect (estimated from the seismic landscape).

3.12. To achieve consistency in the presentation of information, the data should be compiled in a geographical information system with adequate metadata. All data should be stored in a uniform reference frame to facilitate comparison and integration. 3.13. When a seismic hazard evaluation is performed for any reason during the lifetime of the nuclear installation (e.g. for a periodic safety review or a probabilistic seismic hazard analysis for a seismic probabilistic safety evaluation), the existing database should be updated following the recommendations mentioned above as part of the seismic hazard re-evaluation process.

#### GEOLOGICAL, GEOPHYSICAL AND GEOTECHNICAL DATABASE

#### **Regional investigations**

3.14. The purpose of obtaining geological and geophysical data on a regional scale is to provide knowledge of the general geodynamic setting of the region and the current tectonic regime, as well as to identify and characterize those geological features that might influence or relate to the seismic hazard at the site.

3.15. Thus, the extent of the geographical area of interest at a regional scale should be defined in accordance with the recommendations provided in para. 3.4, and by considering the potential sources of all hazards generated by earthquakes that might affect the safety of the nuclear installation(s) at the selected site. The size of the region to be investigated for assessing vibratory ground motion hazards should be large enough to incorporate all seismogenic sources that could affect the nuclear installation: the extent of this region is typically a few hundred kilometres in radius.

3.16. Existing data from any type of published and unpublished geological and geophysical sources (e.g. literature data, country scale data, remote sensing data, data derived from existing galleries, road cuts, geophysical surveys or geotechnical characteristics) should be searched and, if necessary confirmed, by direct observation through geological field reconnaissance visits.

3.17. Where existing data are inadequate to characterize the identified potential geological features relevant to the seismic hazard at the site, and which are identified in the defined region in terms of location, extent and rate of ongoing deformation, a sensitivity analysis should be performed. As result, it may be necessary to verify, first, and complete, later if necessary, the database at a regional scale by acquiring new geological and geophysical data of sufficient detail as is necessary for the geological and geophysical investigations to be conducted in the near region. If needed, identification and analysis of geological and geomorphological evidence (i.e. palaeoseismology, see para. 3.10) of prehistoric and historic earthquakes should also be performed for this purpose.

3.18. The data collected at regional scale should have a resolution consistent with maps to be developed at a scale of 1: 1,000,000, or larger, and with appropriate cross sections. The data should be organized in the project geographical information system within the layer of regional scale information and a summary report should be prepared to describe the studies and investigations performed and results obtained, particularly, in relation to the seismogenic structures identified during this stage of the studies.

#### Near regional investigations

3.19. Site specific geological, geophysical and geotechnical investigations should be conducted in the near region to provide information in more detail than the information available from the regional studies, with the following objectives:

- (a) To define the seismotectonic characteristics of the near region;
- (b) To determine the latest movements of the seismogenic structures identified in the near region;
- (c) To determine the amount and nature of displacements, rates of activity and evidence related to the segmentation of such seismogenic structures.

3.20. The near regional studies should include a geographical area typically not less than 25 km in radius from the border of the site area, although this dimension should be adjusted to reflect local seismotectonic conditions. For new nuclear installation sites for which the exact layout of the buildings and structures have not been defined, the near regional area should be defined from the border of the prospective selected site area boundary.

3.21. These more detailed geological, geophysical and geotechnical investigations should supplement the published and unpublished information already collected for the near regional area, and they should include a definition of the stratigraphy, structural geology and tectonic history of the near region area. The tectonic history should be thoroughly defined for the current tectonic regime, the length of which will depend on the rate of tectonic activity. For example, for studies to assess fault capability, the tectonic information through the Upper Pleistocene to Holocene may be adequate for high seismic regions, while for low seismic regions information through the Pliocene–Quaternary may be necessary.

3.22. In general, for the near regional scale as a whole, the following investigations should be performed in accordance with the procedures and methods established by recognized applicable industry codes and standards. Some of these investigations should be performed specifically in the areas of the identified geological features that might generate potential seismic hazards at the site:

- (a) Geomorphological studies of Quaternary formations or land forms, such as terrace analysis and pedological and sedimentological studies, using well recognized remote sensing image techniques (e.g. aerial and satellite photographs and/or images, Light Detection and Ranging).
- (b) Field geological mapping to identify geomorphology at the scale necessary for the near region studies.
- (c) Subsurface data derived from borehole and geophysical investigations, such as high resolution seismic reflection and/or refraction profiles, and gravimetric, electric and magnetic tomography techniques, to characterize spatially the identified seismogenic structures considered to be relevant in terms of their geometry, extent and rate of deformation. Bathymetry information should also be obtained for geomorphological

investigation in dealing with offshore areas for sites located on or near a coastline. Use of heat flow data may also be necessary.

- (d) Geochronological dating, using recognized, reliable and applicable techniques with appropriate care for stratigraphic purposes.
- (e) Data derived from geodetic methods, such as the global positioning system and interferometry images, and strain rate measurements to assess the ongoing rate and type of tectonic deformation.
- (f) Hydrogeological investigations using new and existing boreholes, wells and other techniques to define the geometry, physical and chemical properties, and steady state behaviour (e.g. water table depth, recharge rate, transmissivity) of all aquifers in the scale necessary for the near regional studies.
- (g) Palaeoseismological and trenching investigations, as needed from the analysis of the data and results obtained by the studies performed as listed in (a) to (f) above.

3.23. Investigations should be made in sufficient detail so that the causes of each geological and geomorphological feature that is relevant (e.g. topographic or structural features as found in aerial photographs, remote sensing imagery or geophysical data) can be properly included in a reasonable model postulated for the recent geological evolution of the area.

3.24. The data collected, and the results obtained from the investigations performed at near regional scale should have a resolution consistent with maps to be developed at a scale of 1:50,000, or larger, and with appropriate cross-sections. Digital elevation models should also be part of the results obtained from this task. The data should be organized in the project geographical information system within the layer of near region scale information. A summary report should be prepared to describe the studies and investigations performed, the evaluation of information for inclusion in the models, and the results obtained, particularly in relation to the seismogenic structures further identified and characterized during this stage of the studies.

#### Site vicinity investigations

3.25. In addition to the information collected at regional and near regional scales, more detailed geological, geophysical and geotechnical studies should be conducted in the site vicinity geographical area with the objective to provide a more detailed database for this smaller area, regarding the definition and characterization in greater detail of the neotectonic history of the identified seismogenic structures (i.e. faults), especially for determining the potential for and the rate of fault displacement at the site (fault capability), and to identify conditions of potential geological and/or geotechnical instability and associated earthquake generated hazards that might affect the nuclear installation.

3.26. Site vicinity studies should cover a geographical area typically not less than 5 km in radius from the border of the nuclear installation site area. For new nuclear installation sites

for which the exact layout of the buildings and structures have not been defined, the 5 km radius area should be defined from the border of the prospective selected site area.

3.27. Geological, geophysical and geotechnical investigations of the site vicinity area should be planned and performed in greater detail than those performed for the near regional scale to be consistent with the tectonic environment and the geological features identified and characterized in previous scale studies (i.e. at regional and near regional scales). In this regard, more detailed geotechnical investigations should be undertaken in the site vicinity scale area, including the drilling of boreholes of an adequate number and depth, and sampling and laboratory testing.

3.28. These detailed investigations should be performed in accordance with the procedures and methods established by recognized applicable industry codes and standards, and as result the following data should be obtained:

- (a) Geological map at the site vicinity scale with cross-sections;
- (b) Age, type, amount and rate of displacement of all the seismogenic structures identified in the area;
- (c) Identification and characterization of locations potentially exhibiting hazards induced by earthquake generated phenomena (e.g. landslide, subsidence, subsurface cavities or karstic processes, failure of dams or water retaining structures).

3.29. The data collected, and the results obtained at site vicinity scale should have a resolution consistent with maps to be developed at a scale of 1:5000, or larger, and with appropriate cross-sections. Digital elevation models should be also part of the results obtained from this task. The data should be organized in the geographical information system within the layer of site vicinity scale information and a summary report should be prepared to describe the studies and investigations performed, the evaluation of information for inclusion in the models, and the results obtained, particularly in relation to the seismogenic structures further identified and characterized during this stage of the studies.

#### Site area investigations

3.30. Additional geological, geophysical and geotechnical seismological site specific studies should be conducted in the nuclear installation site area with the primary objective to provide: (i) detailed knowledge for assessing the potential for permanent ground displacement phenomena associated with earthquakes (e.g. fault capability, liquefaction, subsidence or collapse due to subsurface cavities), and (ii) information on the static and dynamic properties of rock and soil materials beneath the structures foundation (such as P-wave and S-wave velocities, seismic quality factor Q, density) to be used in the specific site response analysis to be performed for assessing the vibratory ground motions that might affect the safety of the structures, systems and components of the nuclear installation.

3.31. As a principle, the site area studies should include the entire area covered by the nuclear installation. For a proposed new site of a nuclear installation, during the site evaluation stage, the exact layout of the units and/or facilities may not yet be known and due to this reason, the entire prospective selected site area should be considered. For the existing site of an operating nuclear installation and for which seismic safety re-evaluation is required, the site area should be well defined. If construction is planned for additional nuclear power plant units to be located in the existing site area, this aspect should be taken into consideration for defining the extent of the site area.

3.32. Detailed geological, geophysical and geotechnical investigations and studies of the site area should be performed in accordance with the procedures and methods established by recognized applicable industry codes and standards, and by using field and laboratory techniques, as follows:

- (a) Geological, geophysical and geotechnical investigations to define the detailed stratigraphy and the structure of the area. Borehole drilling, sampling and/or test excavations (including in situ testing), geophysical techniques and laboratory tests should be performed to determine the thickness, depth, dip, and static and dynamic properties of the different subsurface layers as may be needed by engineering models (e.g. Poisson's ratio, Young's modulus, shear modulus reduction or non-linear properties, density, relative density, shear strength and consolidation characteristics, grain size distribution, P-wave and S-wave velocities).
- (b) Hydrogeological investigations using boreholes and other techniques should be conducted to define the geometry, physical and chemical properties, and steady state behaviour (e.g. water table depth, recharge rate, transmissivity) of all aquifers in the site area, with the specific purpose of determining the stability of soils and how they interact with the foundation of the nuclear installation structures, buildings and components.
- (c) All the data necessary for assessing the specific site response and the dynamic soil– structure interaction analysis should be acquired during these investigations at the site area. For completeness and efficiency, the investigations described should be integrated with the investigations needed for the dynamic soil–structure interaction as described in NS-G-3.6 [3] and NS-G-1.6 [5].

3.33. The data collected at the site area scale are typically presented on maps at a scale of 1:500 and with appropriate cross sections. The data should be organized in the geographical information system within the layer of site area scale information and a summary report should be prepared to describe the studies and investigations performed, the evaluation of that information for inclusion in the models, and the results obtained, particularly in relation to the seismogenic structures and associated seismic hazards further identified and characterized during this stage of the studies.

#### SEISMOLOGICAL DATABASE

3.34. As indicated in para. 3.3, information and data in relation to pre-historical and historical periods is required to be collected for assessing the earthquake related external hazards; this might affect the safety of the nuclear installations at a site due to the need to consider events that occurred with a very long recurrence period (or very low annual frequency of exceedance). To meet such a requirement, the seismological database should include the consideration of past events, that is, of pre-historical and historical earthquake events that have occurred in the region and which might have had the potential to generate seismic hazards at the site.

3.35. The temporal scales of the investigations should be divided as follows:

- a) Pre-historical stage, i.e. the period that is not described in written documents. It includes the period in which earthquake evidence might only be retrieved from archaeological sites as described in carvings, paintings, monuments, drawings and other artefacts, including palaeoseismological evidence.
- b) Historical stage, i.e. the period that is described in written documents, including also palaeoseismological and archaeological evidence. This period is further subdivided as:
  - b.1. Pre-instrumental (or non-instrumental) period;
  - b.2. Instrumental period, i.e. the period from the development and use of instruments to record earthquake parameters.

3.36. A specific 'Project Earthquake Catalogue' should be developed as result of the seismological investigations and as an end-product of the seismological database, including all earthquake related information developed for the project covering all the temporal scales defined in para. 3.35.

#### Pre-historical and pre-instrumental historical earthquake data

3.37. All pre-historical and pre-instrumental historical data on earthquakes should be collected extending as far back in time as possible. Palaeoseismic and archaeo-seismological information on historical and prehistoric earthquakes should also be collected for such purposes.

3.38. To the extent possible, the information on each earthquake in the database should include information on:

- (a) Date, time and duration of the event;
- (b) Location of the macroseismic epicentre of the event;
- (c) Estimated focal depth of the event;

- (d) Estimated magnitude of the event, including the type of magnitude (e.g. moment magnitude, surface wave magnitude, body wave magnitude, local magnitude or duration magnitude; see Definitions), documentation of the methods used to estimate magnitude from the macroseismic intensity and the estimated uncertainty in the magnitude estimate;
- (e) Maximum intensity and, if different, intensity at the macroseismic epicentre, with a description of local conditions and observed damage;
- (f) Isoseismal contours of the event;
- (g) Intensity of the earthquake at the nuclear installation site, together with any available details of effects on the soil and the landscape;
- (h) Estimates of uncertainty for all the parameters mentioned above;
- (i) An assessment of the quality and quantity of data based on which such parameters have been estimated;
- (j) Information on felt foreshocks and aftershocks;
- (k) Information on the causative fault.

The intensity scale used in the Project Earthquake Catalogue should be specified (i.e. because intensity levels can vary, depending on the scale used). The estimates of magnitude and depth for each earthquake should be based on relevant empirical relationships between instrumental data and macroseismic information, which may be developed from the database directly from intensity data or by using isoseismals.

#### Instrumental historical earthquake data

3.39. All available instrumental earthquake data should be collected. Existing information on crustal models should be obtained in order to locate the epicentres of earthquakes.

3.40. Where sufficient information exists, the data to be obtained for each earthquake in the database should include:

- (a) Date, duration and time of origin of the event;
- (b) Coordinates of the epicentre;
- (c) Focal depth of the event;
- (d) All magnitude determinations, including those on different scales;
- (e) Information on observed or recorded foreshocks and aftershocks;

- (f) Other information that may be helpful for understanding the seismotectonic regime, such as focal mechanism, seismic moment, stress drop and other seismic source parameters;
- (g) Macroseismic details;
- (h) Fault rupture inhomogeneity such as asperity (or the strong motion generation area) location and size;
- (i) Estimates of uncertainty for each of the parameters mentioned;
- (j) Information on the causative fault including geometry (length width, strike, dip and rake angles), directivity and duration of rupture;
- (k) Records from both broadband seismometers and strong motion accelerographs.

3.41. Wherever possible, available recordings of regional and local strong ground motion should be collected and used for deriving appropriate ground motion characteristics as discussed in Section 9.

#### **Project Earthquake Catalogue**

3.42. For a proposed new site of a nuclear installation, a specific Project Earthquake Catalogue should be developed for the entire regional area through four major stages: (i) catalogue compilation, (ii) assessment of a uniform size measure to apply to each earthquake, (iii) identification of dependent earthquakes (catalogue declustering), and (iv) assessment of the completeness of the catalogue as a function of location, time and earthquake size. For sites with existing nuclear installations for which earthquake catalogues are already available, they should be updated to reflect the newly collected data and information as well as newly available methods.

3.43. When the site specific catalogue of raw prehistorical and historical (including preinstrumental and instrumental) earthquake data has been compiled, an assessment of the completeness and reliability of the information it contains, particularly in terms of macroseismic intensity, magnitude, date, location and focal depth, should be conducted in order to ensure the record of the occurrence of all known earthquakes in the magnitude range considered important to characterize future seismic hazards. In general, the database is incomplete for small magnitude events owing to the threshold of recording sensitivity, and it is also incomplete for large magnitude events owing to their long recurrence intervals (and the comparatively short period of coverage of the catalogues). Appropriate methods should be used to take account of this incompleteness. In general, different periods of completeness should be identified using statistical methods and considering historical and social context.

3.44. When existing catalogues are incorporated, and data is transferred from these catalogues to the site specific Project Earthquake Catalogue, care should be taken in the merging of

existing catalogues to avoid duplication of events; care should also be taken when establishing the priorities for considering one data point preferable to another.

3.45. If the seismic hazard analysis necessitates that the database is to be composed of independent events (i.e. Poissonian), then a declustering analysis should be performed to identify and separate foreshocks and aftershocks.

3.46. The uncertainties related to the parameters indicated in the data relating to pre-historical and historical periods should be identified and quantified to the extent possible. These uncertainties should be indicated as an entry in the catalogue.

3.47. As a summary, prior to the use of the Project Earthquake Catalogue to either estimate the magnitude–frequency relationship for a seismic source, or to estimate the potential maximum magnitude value for each seismic source, a thorough evaluation and data processing of the catalogue should be performed. This should include:

- (a) Selection of a consistent magnitude scale for use in the seismic hazard analysis;
- (b) Determination of the uniform magnitude of each event in the catalogue on the selected magnitude scale;
- (c) Identification of main shocks (i.e. declustering of foreshocks and aftershocks);
- (d) Estimation of completeness of the catalogue as a function of magnitude, regional location and time period;
- (e) Quality assessment of the derived data, with uncertainty estimates of all parameters.

3.48. The magnitude scale selected for the catalogue should be consistent with the magnitude scale used in the GMPEs that are used in the vibratory ground motion hazard calculations. In deriving magnitude–frequency relationships, the selected magnitude scale should vary almost linearly with the moment magnitude ( $M_w$ ) scale across the magnitude range of interest, to avoid magnitude saturation effects. This is consistent with the use of  $M_w$  becoming a worldwide standard, owing to its increased use in seismology and the development of GMPEs.

3.49. A magnitude–frequency relationship should be developed for each seismic source. Each magnitude–frequency relationship should include the maximum potential magnitude for which the magnitude–frequency relationship applies.

3.50. Uncertainty in the parameters of the magnitude–frequency relationship should be defined by probability distributions that account for any correlation between the parameters.

#### Site specific instrumental data

3.51. To acquire more detailed information on potential seismic sources, path effects, Green's function, ground motion prediction equation testing and a better characterization of the site response, a seismic monitoring network of high sensitivity seismometers, having a recording

capability for micro-earthquakes and capable of recording sufficiently high frequencies to estimate near surface attenuation, should be installed and operated at the site near-region of the nuclear installation. The data obtained from the operation of this network should be also used as a supporting tool in decisions regarding the capability of faults (see Section 7).

3.52. The seismic monitoring network of high sensitivity seismographs should be installed for new sites from the very beginning of the site evaluation stage. For existing sites, for which such networks were not originally deployed, the seismic monitoring network should be installed from the beginning of the seismic safety re-evaluation programme. This network should operate during the whole lifetime of the nuclear installation.

3.53. It is advisable to link the operation and data processing of this seismic monitoring network to any existing regional and/or national seismic networks.

3.54. If the selected instrumentation for the seismic monitoring network cannot adequately record strong motions, several strong motion accelerometers should be collocated with the high sensitivity seismometers.

3.55. Earthquakes recorded within and near the seismic monitoring network should be carefully analysed in connection with seismotectonic studies of the near region.

3.56. The instrumentation should be appropriately and periodically upgraded and calibrated to provide adequate information in line with updated international practices. A maintenance programme, including data communication aspects, should be in place to ensure that no significant lapses occur.

#### 4. CONSTRUCTION OF SEISMIC SOURCE MODELS

#### **GENERAL**

4.1 This section provides recommendations for constructing the models that should properly represent the seismic sources that can generate seismic hazards at the site, which have been identified and characterized based on the data and information compiled and collected as recommended in Section 3. These models should be used for performing the site-specific seismic hazard assessment.

4.2 The link between the integrated geological, geophysical, geotechnical and seismological database and the calculation of the seismic hazard is a seismic source model, which should be based on a coherent merging of the individual databases including due consideration of the available seismotectonic models that may exist or be postulated at regional general scale. The seismic source model constitutes the conceptual and mathematical representation of the physical nature of the seismic sources identified based on the information compiled in the indicated databases and seismotectonic models. One or several seismic source models can be postulated. In the construction of such models, all relevant interpretations of the available data should be taken into account with due consideration of all involved uncertainties. These models include the detailed characterization of the seismic sources and they should be constructed to be used specifically for the seismic hazard analysis applying either deterministic or probabilistic approaches.

4.3 The process for constructing the seismic source models starts with the integration of the elements of seismological, geophysical, geological and other relevant databases into an integrated database, as recommended in Section 3, to obtain a coherent model (and potential alternative models). This integrated database should also include the available seismotectonic models that at regional scale contains the geographic area of interest for assessing the seismic hazards at the specific nuclear installation site. These seismotectonic models should also include considerations on the uncertainties embedded either expressly or implicitly in their characterization.

4.4 Based on the available data and information included in the integrated database and on the interpretations provided by the correspondingly involved experts, a detailed characterization of all identified and postulated seismic sources should be conducted with the aim of identifying and characterizing in detail all sources of earthquakes which may contribute to the seismic hazard at the site. This source characterization provides all the necessary characteristics (e.g. location and geometries, maximum magnitude and recurrence) of the identified seismic sources.

4.5 The seismogenic structures identified throughout the process of compiling the database might not explain all the observed earthquake activities. This is because seismogenic structures may exist without recognized surface or subsurface manifestations, and also because of the timescales involved; for example, fault displacements may have long

recurrence intervals with respect to seismological observation periods. Consequently, the seismic source model(s) should consist, to a greater or lesser extent, of two types of seismic sources:

- 1) Those seismogenic structures that can be identified and characterized by using the available database;
- 2) Diffuse seismicity (consisting usually, but not always, of small to moderate earthquakes) that is not attributable to specific seismogenic structures that might be identified by using the available database.

4.6 The identification and characterization of seismic sources of both types should include assessments of the specific uncertainty involved in each type. Diffuse seismicity poses a particularly complex problem in seismic hazard evaluation and will generally involve greater uncertainty because the causative faults of earthquakes are either not well understood or are not well characterized with currently available information.

4.7 The construction of the seismic source model(s) and the characterization of all parameters of each of their elements should be based primarily on interpretation and evaluation of the available data.

4.8 If the compiled geological, geophysical and seismological data support alternative seismic source models, and the differences in these models cannot be resolved by means of additional investigations within a reasonable time frame, all such models should be taken into consideration in the final hazard evaluation.

4.9 The validity of the proposed seismic source models based on these should be tested against existing knowledge and information, for example, by comparing long term strain rates predicted by the model against available and reliable geodetic and geological observations.

#### SEISMOGENIC STRUCTURES (IDENTIFIED SEISMIC SOURCES)

#### Identification

4.10 All seismogenic structures that might contribute to the seismic hazards at the site should be included in the constructed seismic source model(s) and supported by sensitivity analysis.

4.11 Regarding the fault displacement hazard evaluation, special attention and consideration should be given to those seismogenic structures close to the site that have a potential for primary and/or secondary displacement at or near the ground surface (i.e. capable faults, see Section 7).

4.12 The identification of seismogenic structures should consider those geological features for which direct or indirect evidence exists of there having been a seismic source within the current tectonic regime.

4.13 When specific data on a particular geological feature are insufficient for its detailed characterization, a detailed comparison of this feature with other analogous geological features in the region, or in similar tectonic regions in the world, should be made in terms of their age of origin, sense of movement and history of movement, to help determine whether the feature can be considered a seismogenic source.

#### Characterization

4.14 For seismogenic structures that have been identified as being relevant to determining the earthquake generated hazards for the site, the associated characteristics of such structures should be determined. The structure (length, depth, width), orientation (strike, dip), rake of dislocation, rate of deformation, and geological complexity (segmentation, branching, secondary faults) should be determined to the extent possible. These characteristics should be determined based upon evaluation of all data and information contained in the geological, geophysical, geotechnical and seismological databases.

4.15 Available information about the seismological and geological history of the movement of a fault or structure (such as segmentation, fault length, and fault width) should be used to estimate the maximum rupture dimensions and/or displacements. This information together with magnitude-area scaling relationships should be used to evaluate the maximum potential magnitude of the seismogenic structure under consideration. Other data that may be used to construct a rheological profile should also be considered in this estimation, such as data on heat flow, crustal thickness and strain rate.

4.16 In locations where a fault zone comprises multiple fault segments, each fault segment should be taken into account both dependently and independently. The possibility of the multiple fault segments rupturing simultaneously during an earthquake should also be evaluated. In order to determine the best estimate and associated uncertainties of the maximum potential magnitude, a suite of possible total fault rupture length scenarios should be developed.

4.17 The maximum potential magnitude  $(m_{max})$  associated with each seismic source should be specified, and the uncertainty in  $m_{max}$  should be described by a discrete or continuous probability distribution. For each seismic source, the value of  $m_{max}$  is used as the upper limit of integration in a probabilistic vibratory ground motion hazard calculation to derive the magnitude–frequency relationship, and as the scenario magnitude in a deterministic vibratory ground motion hazard evaluation. In general, but especially for sites in intraplate settings, the largest observed earthquake is a poor and unconservative estimate of  $m_{max}$ . The use of global analogues should be considered, and care should be taken to determine the appropriate seismotectonic analogue. The sensitivity of the resulting hazard to the selection of the  $m_{max}$  distributions should be tested.

4.18 Other approaches that are available for estimating maximum potential magnitudes based on the statistical analysis of the magnitude–frequency relationships for earthquakes associated with a particular structure should also be considered, as appropriate. These approaches assume an association between the structure and all the earthquake data used. In all cases, the results of these methods should be confirmed to be consistent with the available collected data including palaeoseismological data.

4.19 Regardless of the approach or combination of approaches used, the determination of the maximum potential magnitude might have significant uncertainty, which should be incorporated into the analysis to the extent that it is consistent with seismological, geological and geomorphological data.

4.20 In addition to the maximum potential magnitude, a magnitude–frequency relationship should be derived for each seismogenic structure included in the seismic source model, to determine: (a) the rate of earthquake activity; (b) an appropriate type of magnitude–frequency relationship (e.g. characteristic or exponential); and (c) the uncertainty in this relationship and in its parameters. In the case of the characteristic earthquake occurrence model, the last event should be identified as far as possible.

4.21 For those seismic sources that have registered the occurrence of few earthquakes in the compiled geological and seismological databases, the determination of magnitude–frequency relationships (e.g. the Gutenberg–Richter relationship) may involve a different approach, which may include adopting a value that represents the regional tectonic setting of the seismic source; for example, a stable continental tectonic setting. This approach is viable because many studies have shown that the *b* value of the Gutenberg–Richter relationship varies over a relatively narrow range within a given tectonic setting. Regardless of the approach used to determine the *a* and *b* values of the magnitude–frequency relationship, the uncertainty in those parameters and their correlations should be appropriately assessed and incorporated into the seismic hazard analysis.

#### ZONES OF DIFFUSE SEISMICITY

#### Identification

4.22 Zones of diffuse seismicity are those areas in which there is evidence of seismicity that it is not attributable to any specific identified seismogenic structures based on the available databases and seismotectonic models. Each zone should be assumed to encompass an area that possesses similar seismotectonics.

4.23 In the performance of a seismic hazard evaluation, knowledge about the depth distribution of the diffuse seismicity (e.g. derived from the seismological, geological and geophysical databases) should be incorporated and the thickness and edepth of the seismogenic zone should be properly characterized.

4.24 Significant differences in rates of earthquake occurrence may suggest different tectonic conditions and they should be considered in defining the boundaries of the zone of diffuse seismicity. Significant differences in focal depths (e.g. crustal versus subcrustal), focal mechanisms, states of stress, tectonic characteristics and Gutenberg–Richter b values may all be used to differentiate between diffuse seismicity zones.

#### Characterization

4.25 The  $m_{max}$  associated with zone of diffused seismicity should be evaluated based on seismological data and the seismotectonic characteristics of the diffuse seismicity zone. Comparison with similar world regions for which extensive seismological data are available may be useful, but informed judgement should be used in such an evaluation. Often the value of  $m_{max}$  obtained will have significant uncertainty owing to the relatively short time period covered by the seismological data with respect to the processes of ongoing deformation. This uncertainty should be appropriately represented in the seismic source model.

4.26 Available information about the seismological and geological history of the seismotectonic structure (such as stress regime, strain rate, etc.) should be used to estimate the  $m_{max}$ . Other data that may be used to construct a rheological profile should also be considered in this estimation, such as data on heat flow, crustal thickness and micro-earthquake distribution.

4.27 The  $m_{max}$  associated with each seismic source should be specified, and the uncertainty in  $m_{max}$  should be described by a discrete or continuous probability distribution. For each seismic source, the value of  $m_{max}$  is used as the upper limit of integration in a probabilistic vibratory ground motion hazard calculation to derive the magnitude–frequency relationship, and as the scenario magnitude in a deterministic vibratory ground motion hazard evaluation. In general, but especially for sites in intraplate settings, the largest observed earthquake is a poor and unconservative estimate of.  $m_{max}$ . The use of global analogues should be considered, and care should be taken to determine the appropriate seismotectonic analogue. The sensitivity of the resulting hazard to the selection of the  $m_{max}$  distributions should be tested.

4.28 Other approaches that are available for estimating  $m_{max}$  based on the statistical analysis of the magnitude–frequency relationships for earthquakes associated with a particular structure should also be considered, as appropriate. These approaches assume an association between the structure and all the earthquake data used. In all cases, the results of these methods should be confirmed to be consistent with the available collected data including palaeoseismological data.

4.29 Regardless of the approach or combination of approaches used, the determination of the  $m_{max}$  might have significant uncertainty, which should be incorporated into the analysis to the extent that it is consistent with seismological, geological and geomorphological data.

4.30 In addition to the  $m_{max}$ , a magnitude–frequency relationship should be derived for each seismogenic structure included in the seismic source model, to determine: (a) the rate of earthquake activity; (b) an appropriate exponential magnitude–frequency relationship (e.g. Gutenberg–Richter relationship); and (c) the uncertainty in this relationship and in its parameters.

4.31 For those seismic sources that have registered the occurrence of few earthquakes in the compiled geological and seismological databases, the determination of magnitude–frequency relationships (e.g. the Gutenberg–Richter relationship) may involve a different approach, which may include adopting a value that represents the regional tectonic setting of the seismic source;

for example, a stable continental tectonic setting. This approach is viable because many studies have shown that the b value varies over a relatively narrow range within a given tectonic setting. Regardless of the approach used to determine the a and b values of the magnitude–frequency relationship, the uncertainty in those parameters and their correlations should be appropriately assessed and incorporated into the seismic hazard analysis.

#### 5. VIBRATORY GROUND MOTION ESTIMATION METHODS

#### GENERAL

5.1 This section provides recommendations on the methods for estimating the vibratory ground motion at the nuclear installation site.

5.2 The variability associated with the prediction of the vibratory ground motions from future earthquakes is typically one of the largest sources of uncertainty in seismic hazard assessment. Currently available methods for estimating ground motions include GMPEs, which are primarily empirical, and direct simulation methods, which are physic-based scaling to interpolate a smaller range of data. These alternative methods are described in the subsections below. Given the significant epistemic uncertainty currently inherent in ground motion prediction, multiple relationships and/or methodologies should be utilized. However, the evaluation of ground motion using different methods should be done in a consistent and complementary manner.

5.3 Individual models for the prediction of vibratory ground motions should include both an estimate of the median ground motion amplitude which is the mean of logarithmic normal distribution, as well as a measure of the aleatory variability about the mean. The final complete vibratory ground motion model should include an assessment of the epistemic uncertainty in both the mean prediction as well as its aleatory variability in the logarithmic scale.

5.4 The definition of the vibratory ground motion intensity measure used in the ground motion characterization should be consistent with the intended use in subsequent engineering design and probabilistic safety analyses for structures, systems and components of the nuclear installation and for the assessment of ground failures such as slope failures and liquefaction. Empirical relationships are typically developed for horizontal response spectral acceleration at 5% of critical damping. Alternative damping levels can be derived using published scaling relationships. Simulation methods typically produce ground motion time histories from which any necessary intensity measure can be derived directly.

5.5 Care should be taken to ensure that the way in which the horizontal components of ground motion are represented in the chosen GMPEs is consistent with the subsequent engineering use in design or fragility analyses. The number of spectral periods characterized should be sufficient to develop smooth spectral shapes (see Section 8).

5.6 The vibratory ground motion should be calculated at a specific location within the soil profile of the nuclear installation site, which is indicated as the control point. In some situations, multiple control points may be necessary. The specification of the control point is an important interface issue and should be clearly defined from the beginning of the project in accordance with user requirements (see Section 10). The control point location could be defined at the free field ground surface, at the outcrop of bedrock, or at any other specified depth in the soil profile. From the vibratory ground motion specified at the defined control point, the vibratory ground motion to be used as the input for calculating the response of the

structures, systems and components of nuclear installations should be evaluated and developed through an appropriate site response analysis.

#### **GROUND MOTION PREDICTION EQUATIONS**

#### **Selection criteria**

5.7 Ground motion prediction equations (GMPEs) specify the median value of vibratory ground motion amplitude based on a limited number of explanatory variables such as earthquake magnitude, distance, style of faulting, geometry, and site conditions. The model may be in the form of an equation or a table. Even for models that are primarily based on empirical data, simulation results are often used to provide constraints on scaling behaviour for magnitudes, distances or geometries that are not well-represented in the existing databases. Typically, a set of GMPEs are selected and used for performing the seismic hazard analysis.

5.8 The selection of the set of appropriate GMPEs should be based on their consistency with the seismotectonic conditions and with the output parameters needed for the seismic hazard analysis recommended in Section 10. The range of magnitudes, distances and other parameters for which the GMPE is valid should be checked.

5.9 The selection of candidate GMPEs to be used in the seismic hazard assessment should be based on the following general criteria:

- (a) They should be current and well established, supported by an adequate quantity of properly processed data;
- (b) They should be determined by appropriate regression analysis to avoid that an error on a subjectively fixed coefficient will propagate to the other coefficients.
- (c) They should be consistent with the types of earthquakes and the attenuation characteristics of the site region;
- (d) They should match as closely as possible the tectonic environment of the site region;
- (e) They should make use of available local ground motion data as much as possible;
- (f) They should be consistent with the physical characteristics of the control point location.

5.10 In active tectonic regions, relatively abundant empirical data exists and GMPEs should be developed primarily from that data or from data from similar seismotectonic settings. In areas with lower rates of earthquake activity, where data is much less abundant (such as stable continental regions), alternative empirical or semi-empirical methods have been developed for deriving GMPEs. Examples of these methods include the hybrid empirical method and hybrid reference empirical method, both of which rely on utilizing a GMPE developed for regions where abundant data exist (a host region). In the hybrid empirical method, simple parametric seismological models of the physical properties of the seismic source and diminution of seismic energy with distance are used to adjust the host GMPE to conditions consistent with the site or region of interest (the target conditions). For the hybrid reference empirical method, adjustments should be developed based on residuals between the empirical data in the target region and the GMPE model from the host region. This approach requires an adequate number of empirical data in the target region to perform the necessary residual analysis for the development of the adjustments.

5.11 If adequate data do not exist in the site region to directly develop a reliable suite of GMPEs then the adjustments described in para. 5.10 should be used to adjust well-calibrated GMPEs from other regions so they satisfy the general criteria in para. 5.9. To avoid the error propagation among coefficients in the GMPEs, the coefficients should be evaluated based on physics-based scaling. If non-ergodic GMPEs are to be used, all coefficients should be properly identified to represent the ground motions for the specific conditions. If ergodic GMPEs are to be used, they should be able to capture overall ground motion characteristics with less parameters, although it is known that the standard deviation may be larger than the non-ergodic GMPEs.

5.12 The aleatory variability should be considered for the GMPEs and derived from the residuals between observed and predicted motions. The residuals may depend on magnitude distance or ground motion level itself. At the selected specific site, detailed site response analysis or the residual investigation using vibratory ground motions recorded at the site should be conducted in order to reduce the aleatory variability.

5.13 Empirically derived vertical vibratory ground motion should be represented either as a vertical-component GMPE or as an empirically derived ratio between vertical and horizontal components of motion. Caution should be exercised when utilizing a vertical-component GMPE in the seismic hazard assessment calculations as the scenario earthquakes that are developed might differ from those derived for the horizontal case.

5.14 Caution should be exercised in comparing the selected GMPEs with recorded ground motions from small, locally recorded earthquakes. The use of such recordings (e.g. in scaling the selected attenuation relationships) should be justified by showing that their inferred magnitudes and distance scaling properties are appropriate for earthquakes within the ranges of magnitude and distance that are of greatest concern regarding the seismic safety of the nuclear installation. Nevertheless, best efforts should be performed to reflect those observed data in the selection of the GMPEs.

5.15 When available, macro seismic intensity data should also be used to assign weights to GMPEs or calibrate the selected GMPEs in those regions where instruments for recording strong motion have not been in operation for a long enough period to provide sufficient amounts of instrumental data. These data may be used at least in a qualitative manner to verify that the GMPEs used to calculate the seismic hazard are representative of the regional ground motion characteristics. However, care should be exercised when performing these

comparisons as the uncertainty in translating macro-intensity data to the desired ground motion intensity metric can be significant.

#### Epistemic uncertainties of the technically defensible interpretations

5.16 The appropriate treatment of epistemic uncertainties requires the identification, evaluation, and quantification of the range of possible vibratory ground motions that might occur at a site. Except for regions where a sufficient number of independent, region-specific GMPEs have been published, the full quantification of the range of possible ground motions might not be possible using the selection of GMPEs currently available for a specific region. Epistemic uncertainties should be quantified. This would require using models from other regions and applying adjustments (as described in paragraphs 5.10 and 5.11) either to render the models more applicable to local conditions or to make the models compatible in terms of predictor variables.

5.17 There are several alternative methodologies that should be used to represent the centre, body and range of technically defensible interpretations for estimating ground motions at a site from future earthquakes. All methods begin with the development of a representative suite of GMPEs that satisfy the selection criteria described in para. 5.9. The methodologies to develop weights for individual GMPEs based upon the degree of confidence in each GMPE and/or approach and the conformance with existing data. Consideration should be given in the application of this approach to developing a representation of the future median ground motions that is a complete and representative (unbiased) sample of the ground motion space.

#### GROUND MOTION SIMULATION METHODS

5.18 Ground motion simulations provide results that can be used to refine and calibrate empirical GMPEs, to directly develop ground motion prediction models, and to develop ground motions for specific scenario events. Several simulation methods exist and are described in following paragraphs. Any simulation approach, if used, should be carefully validated and calibrated against available empirical data.

5.19 One commonly used approach utilizes a stochastic simulation methodology based on simple parametric models that represent the physical properties of the seismic source and propagation and attenuation of seismic energy. This methodology can either represent the source as a point or as a finite fault with rupture that evolves in space and time. Both methodologies should include the development of region specific parametric models for the seismic source, path and site effects that should be calibrated with empirical data from the region of interest.

5.20 Alternative ground motion simulation methods utilize a more direct physical representation of the seismic source and wave propagation. These 'physics-based' methods use fault rupture modelling and path-specific wave propagation to estimate ground motions. These procedures may be especially effective in cases where nearby faults contribute significantly to the vibratory ground motion hazard at the site and/or where the existing empirical data is limited (on the hanging wall of a nearby fault for example). The physics-based methods fall

into two general categories, kinematic and dynamic. The kinematic simulation method should specify the following parameters:

- (a) Fault geometry parameters (location, length, width, depth, dip, strike);
- (b) Macro-parameters (hypocenter, seismic moment, average dislocation, rupture velocity, average stress drop);
- (c) Micro-parameters (rise time, dislocation, stress parameters for finite fault elements);
- (d) Subsurface structure parameters, such as shear and compressional (alternatively, Poisson's ratio) wave velocities, density and anelastic attenuation factor (i.e. seismic quality factor Q).

5.21 Most of the parameters mentioned above cannot be known in advance for future ruptures on a specific fault. Hence the simulations should represent these properties as random variables with appropriate correlation amongst some of the variables. The specific characteristics of the seismotectonic setting where the site is located should also be given due consideration. A sufficient number of simulations should be conducted to provide a stable estimate of the median ground motions at the site of interest as well as the variability about that median. Kinematic models typically utilize a stochastic approach to model the high frequency portion of the spectrum.

5.22 In the dynamic simulation approach, the state of stress and the friction law properties on the fault should be defined by, for example, slip weakening friction models that are characterized by the dynamic stress drop, strength excess and critical slip distance distribution on the finite fault. As with the kinematic simulation approach, these properties are unknown for future earthquakes on a specific fault and should be treated as randomly correlated variables.

5.23 If recordings of earthquakes exist at or near the site, this data should be used to either calibrate the theoretical Green's function or used directly as an empirical Green's function in the range of frequencies with high signal to noise ratio.

5.24 Potential inhomogeneity of the fault rupture model should be considered such that a high frequency component and pulse-like signal of the seismic wave could depart from any specific area on the fault. Caution should be taken that high frequency and low frequency components are not always generated from the same area. Furthermore, any available relevant 2-D or 3-D crustal structure model that deviates from the assumption of homogeneous horizontal layered models should be considered for a more realistic wave propagation simulation.

### 6. VIBRATORY GROUND MOTION SEISMIC HAZARD ANALYSIS

#### GENERAL

6.1 This section provides recommendations for performing the vibratory ground motion hazard analysis based on (i) the approaches currently recognized by engineering and regulatory practices and (ii) the data, models and methods for determining vibratory ground motion as described in Sections 3, 4 and 5, respectively.

6.2 The approach to be used for assessing the vibratory ground motion hazard at the nuclear installation site should be defined at the beginning of the seismic hazard evaluation project. The vibratory ground motion hazard may be evaluated by using probabilistic and/or deterministic methods of seismic hazard analysis. The choice of the approach depends on the national regulatory requirements and the end user specifications, which should be documented in the project quality plan (see Section 10).

6.3 The vibratory ground motion seismic hazard analysis should use all the elements and parameters of the postulated seismic source model(s) (see Section 4), including the quantified uncertainties. Alternative models proposed by the experts of the seismic hazard analysis team, should be formally included in the hazard computation.

6.4 In the vibratory ground motion hazard evaluation, both types of uncertainties — aleatory and epistemic — should be considered regardless of the approach used.

6.5 Computer codes that are used in the evaluation of the vibratory ground motion hazard should be able to accommodate the various ground motion prediction and seismic source models defined by the project team for the seismic hazard evaluation for use in the calculations. It should also be demonstrated that these codes account appropriately for the treatment of uncertainties.

6.6 Consideration should be given during the hazard analysis to the appropriate interfacing of the assessed vibratory ground motion and the site soil response analysis.

6.7 The design basis may be derived using either a probabilistic or a deterministic approach, the probabilistic safety assessment of the nuclear installation can only be performed using the results of a probabilistic seismic hazard assessment. Requirements for the use of probabilistic safety assessment for nuclear power plants are established in IAEA Safety Standards Series No. SSR-2/1 (Rev. 1), Safety of Nuclear Power Plants: Design, Specific Safety Requirements [7].

#### PROBABILISTIC SEISMIC HAZARD ASSESSMENT

6.8 A probabilistic approach should be used when the safety of the nuclear installation against earthquake loading needs to be demonstrated with explicit consideration of the likelihood of occurrence of the relevant seismic hazards (e.g. vibratory ground motion level). Probabilistic approaches consider the rates of recurrence of events along with their estimated

maximum size. In these cases, the annual frequency of exceedance of different levels of the relevant hazard parameters (e.g. the peak ground acceleration) should be estimated to define an appropriate design basis and/or to perform a seismic probabilistic safety assessment. In subsequent analyses these results may be used to demonstrate the nature of cliff edge effects and to ensure that performance targets are met.

6.9 The evaluation of the vibratory ground motion seismic hazard by probabilistic methods should include the following steps:

- 1) Select the level of effort, resources and details to be applied in the seismic hazard assessment project considering the safety significance of the nuclear installation, the technical complexity and the uncertainties in the hazard inputs, regulatory requirements and oversight, the amount of contention within the related scientific community, the degree of public concern and the availability of project resources.
- 2) Develop a detailed work plan with careful consideration of the experts that will constitute the project team, and the project reviewers who will participate in the independent peer review process. The work plan should consider the conduct of technical meetings to be held with participation of experts from the project team and from the review team to discuss topics related to (i) issues relating to the hazard determination and the availability and quality of the compiled data, (ii) alternative interpretations, (iii) feedback for the project execution.
- 3) Compile the integrated geological, geophysical and geotechnical database, as recommended in Section 3 and build the seismic source model(s) for the site region in terms of the defined seismic sources, including uncertainty in their boundaries and dimensions, as recommended in Section 4. A 'zoneless' approach is an alternative scheme to avoid boundary issues but its implementation should be adequately justified.
- 4) Evaluate, for each seismic source identified in the seismic source model(s), the maximum potential magnitude distribution, the rate of earthquake occurrence and the type of magnitude–frequency relationship, together with the uncertainty associated with each evaluation.
- 5) Select the appropriate GMPEs for the site region and assess the uncertainties in both the mean and the variability of the ground motion as a function of earthquake magnitude and distance from the seismic source to site. The physics-based simulation techniques as described in Section 5 are alternative schemes to evaluate the ground motion using a sufficient number of calculated time histories to define the central, body and range of the technically defensible interpretations. The selection and/or adjustment of the GMPEs should be done with consideration of the use in site response analysis, meaning that interaction with step 7) below is needed.
- 6) Build analysis models (logic trees) and perform hazard calculations including sensitivity analysis in a phased approach, starting with a preliminary analysis round, discussion of the preliminary results, feedback from the review meetings, and ending with a final

analysis round that will provide the necessary deliverables defined in accordance with the user needs.

- 7) Perform the site response analysis.
- 8) Elaborate, review and approve the final report including all necessary deliverables.

6.10 The smallest annual frequency of exceedance of interest for which the seismic hazard should be calculated will depend on the eventual use of the probabilistic seismic hazard analysis (i.e. whether for design purposes or for input to a seismic probabilistic safety assessment) and should be indicated in the project plan (see Section 10). This value can be extremely low (e.g.  $10^{-8}$ ) when it is associated with seismic probabilistic safety assessment studies in which a nuclear power plant has a very low core damage frequency or large early release frequency in relation to non-seismic initiators (e.g. for innovative reactors). In such cases, care should be taken to assess the suitability and validity of the database, the seismic source model(s), the GMPEs and the basis for the expert opinions, since uncertainties associated with these elements can significantly bias the hazard results.

6.11 To assist in determining the ground motion characteristics at a site, it is often useful to evaluate the fractional contribution from each seismic source to the total vibratory ground motion seismic hazard by means of a de-aggregation process. Such de-aggregation may be carried out for a target annual frequency of exceedance, typically the value selected for determining the design basis ground motion. The de-aggregation may be performed for at least two ground motion frequency ranges, generally at the low and high ends of the spectrum, which can be used to identify the magnitude–distance pairs that have the largest contribution to the annual frequency of exceedance of the selected ground motion frequency ranges, as well as the input for the site response analysis.

6.12 To stay within the range of seismic magnitudes that is represented by the database used in the derivation of the GMPEs, it is necessary to use a corresponding lower seismic magnitude limit. The practice has been to combine this lower limit consideration with an engineering concept that is linked to a ground motion level from a seismic magnitude below which no damage would be incurred by the structures, systems and components important to safety at the nuclear installation. A seismic magnitude value alone is not the best way of representing damage potential. As an alternative to the use of a magnitude measure, the lower bound motion filter may be specified (in terms of an established damage parameter, such as the cumulative absolute velocity, peak ground velocity, instrumental seismic intensity) in conjunction with a specific value of that parameter for which it can be clearly demonstrated that no significant contribution to damage or risk will occur. The lower bound motion filter should be selected in consultation with the seismic designer and/or the fragility and safety analyst.

6.13 Because of the uncertainties, mainly of an epistemic nature, that are involved at each stage of the hazard assessment process, both the assumptions adopted in previous steps and the overall results obtained from the analysis should be evaluated based on available

observations and data from actual seismic events, with due consideration of the difference between the short period of data availability and the return period usually adopted for seismic design of nuclear installations. This evaluation should be used to check either the consistency of the assumptions or the adequacy of the defined branch of the logic tree, or to assign proper weight in the logic tree.

6.14 The results of the vibratory ground motion hazard analysis using a probabilistic approach should be consistent with the typical output shown in the Annex.

#### DETERMINISTIC SEISMIC HAZARD ASSESSMENT

6.15 A deterministic approach should be used when the protection of the nuclear installation against earthquake loading needs to be ensured through adequate design features using a conservative scenario of the relevant seismic hazards (e.g. a conservative level for the vibratory ground motion hazard). In these cases, conservative values of the key hazard parameters should be estimated to define an appropriate design basis for the nuclear installation in accordance with established safety margins within a defense-in-depth framework. The deterministic approach uses single individual values (i.e. a probability of 1) for key parameters leading for a single value for the result (as defined in IAEA Safety Standards Series No. SSG-3, Development and Application of Level 1 Probabilistic Safety Assessment for Nuclear Power Plants [8]).

6.16 The evaluation of the vibratory ground motion seismic hazard by deterministic methods should include the following steps (it should be noted that the first five steps of this process are essentially the same as those described in para. 6.8 for performing a probabilistic seismic hazard assessment):

- 1) Select the level of effort, resources and details to be applied in the seismic hazard evaluation project considering the safety significance of the nuclear installation, the technical complexity and the uncertainties in the hazard inputs, regulatory requirements and oversight, the amount of contention within the related scientific community, the degree of public concern and the availability of project resources.
- 2) Develop a detailed work plan with careful consideration of the experts that will constitute the project team and the project reviewers who will participate in the independent peer review process. The work plan should consider the conduct of technical meetings to be held with participation of experts from the project team and from the review team to discuss topics related to (i) issues relating to the hazard determination and the availability and quality of the compiled data, (ii) alternative interpretations, (iii) feedback for the project execution.
- 3) Use the seismic source model(s) that was(were) compiled as recommended in Section 3, in terms of the defined seismic sources identified on the basis of tectonic characteristics, the rate of earthquake occurrence and the type of magnitude–frequency relationships including non Poissonian models.

- 4) Evaluate the maximum potential magnitude for each identified seismic source included in the seismic source model(s), to be determined considering the maximum magnitude distribution.
- 5) Select the GMPEs adequate for the region and assess the mean and variability of the ground motion to be obtained as a function of earthquake magnitude and the distance from the seismic source to the site, including the influence of the specific site soil conditions.
- 6) Perform the vibratory ground motion hazard calculation such that:
  - (i) For each seismogenic structure, the maximum potential magnitude should be assumed to occur at the point of the seismogenic structure closest to the site area of the nuclear installation, with account taken of the physical dimensions of the seismic source. When the seismogenic structure is within the site vicinity area, the maximum potential magnitude should be assumed to occur beneath the site.
  - (ii) For zones of diffuse seismicity, the associated maximum potential magnitude should be assumed to occur at the point of the region boundary closest to the site.
  - (iii) In a zone of diffuse seismicity that includes the site of the nuclear installation, the maximum potential magnitude should be assumed to occur at some identified specific horizontal and vertical distance from the site. This distance should be determined based on detailed seismological, geological and geophysical investigations (both onshore and offshore) with the goal of showing the absence of faulting in the site vicinity or, if faults are present, they are characterized with the direction, extent, history and/or rate of movements as well as the age of the most recent movement being characterized as older than the established definition for fault capability (see Section 7). This investigation covers an area that is typically less than the near regional area, up to a maximum of about ten kilometres. The actual distance used in the GMPEs will depend on the best estimate of the focal depths and on the physical dimensions of the potential fault ruptures for earthquakes expected to occur in the seismotectonic province.
  - (iv) Several appropriate GMPEs or, in some cases, simulated ground motions based on fault rupture modellings should be used to determine the ground motion that each of the maximum potential magnitude earthquakes would cause at the site, with account taken of the variability of the ground motion.
  - (v) Ground motion characteristics should be obtained as result of applying the deterministic approach, by implementing the recommendations provided in para 5.4.
- 7) Take into account both aleatory and epistemic uncertainties at each step of the deterministic evaluation, to ensure that the conservative procedure described above has covered all involved uncertainties, while avoiding double counting. This approach

should explicitly assess the adequacy of the treatment of uncertainties with respect to the choices that have been made in the different steps (e.g. the assumption that the maximum magnitude earthquake would be located at the closest location to the site) to get an appropriate confidence level at the end of the process.

- 8) Compare the results of the deterministic and probabilistic assessments. This will help in the interpretation of the relevant annual frequency of exceedance for deterministic seismic hazard analysis. In addition, this will enable the deterministic assessment results to be calibrated against the probabilistic seismic hazard assessment results, allowing some risk and performance insights to be developed. This comparison should also take into consideration the evaluation of the results based on the available observations and data, in the same manner as for probabilistic assessment (see para. 6.11).
- 9) Perform the site response analysis.
- 10) Elaborate, review and approve the final report including all necessary deliverables.

#### SITE RESPONSE ANALYSIS

6.17 Once the vibratory ground motion analysis is done at the selected reference site location and elevation, a site response analysis should be done considering the detailed and specific geophysical and geotechnical information on the soil profiles in the site area. The aim of the analysis is to obtain the vibratory ground motion parameters at the free surface at the top of the soil profile, and/or at other locations in the profile, such as the bottom level of the basemat of selected structures and buildings important to safety.

6.18 If the seismic hazard evaluation is performed for a new site within which the precise location and layout of the nuclear installation is not yet known (including the lack of information of its foundation characteristics), the site response analysis should be performed at one of the following locations:

- (a) At the most likely location of the installation within the site area;
- (b) At a location representative of the general geotechnical characteristics of the site area;
- (c) At a 'mean' location assumed as a place with mean values of the geotechnical characteristics of the soil profile.

6.19 The soil response analysis conducted at this early stage using any of the assumptions above will be considered as a 'preliminary' soil response analysis as needed for defining the seismic hazard design basis that should be followed later by a 'final' site response analysis to be performed at the finally defined location of the structures of the nuclear installation. It is also possible to defer the site response analysis until the exact location of the structures of the nuclear installation and their foundation parameters are sufficiently well known.

6.20 If the site is an existing site with operating nuclear installations or a site where the specific type of installation is adequately defined in location and layout, the site response analysis should proceed specifically for such installations.

6.21 Two approaches can be taken to properly consider the specific geological and geotechnical soil conditions at a site as part of the estimation of the seismic vibratory ground motion. The first approach is to utilize GMPEs appropriate for the specific site soil conditions, i.e. using GMPEs that have been developed for subsurface conditions of the type that prevails at the site. The second approach is to conduct a site response analysis compatible with the detailed and specific geotechnical and dynamic characteristics of the soil and rock layers at the site area. The decision on which approach to be used should therefore be made based on the GMPEs utilized for calculating the seismic vibratory ground motion parameters at the site.

6.22 If the first approach described in para. 6.20 is utilized, the resulting vibratory ground motion parameters at the free surface of the top of the soil profile may be used directly for defining the seismic hazard design basis for the nuclear installation. If the second approach is utilized, a step-by-step procedure should be applied as follows:

- (1) Determine the best estimate soil profile parameters based on the geophysical and geotechnical database compiled as recommended in Section 3, for the full depth from the bedrock outcrop layer to the free surface including uncertainties, evaluated in accordance with the value of such depth. This means determining the mean values and associated uncertainties of the following parameters for each site soil layer:
  - (i) The low strain shear wave velocity  $(V_S)$ ;
  - (ii) The strain dependent shear modulus reduction and hysteretic damping properties;
  - (iii) The soil density;
  - (iv) The layer thickness;
  - (v) For the vertical component, the compressional wave velocity  $(V_P)$ .
- (2) Evaluate the correlation of soil layer properties, i.e. determine whether they correspond at the same time for each layer so that their characteristics should be correlated or uncorrelated in the simulations.
- (3) Determine whether analyses other than equivalent linear analyses should be performed; if so, analysis using non-linear properties of the soil should be necessary.
- (4) Starting with the seismic hazard curves obtained at the bedrock outcrop layer, calculate site amplification factors through convolution of the bedrock hazard curves for each spectral frequency of interest, so that they mimic the characteristics of the principle contributors to the de-aggregated seismic hazard, including diffuse seismicity.
- (5) Develop the uniform hazard response spectra at the identified locations of interest for the nuclear installation site and for the annual frequencies of exceedance selected for defining the seismic design basis (e.g. 10<sup>-4</sup> and 10<sup>-5</sup> per year). Note that the final design basis ground motion should be developed with enough safety margin beyond this level.
- (6) Verify the site response analysis results with any available observed instrumental

records).

(7) If the subsurface structure and buried foundations are complex, soil-structure interaction analysis should be conducted.

## 7. EVALUATION OF THE POTENTIAL FOR FAULT DISPLACEMENT AT THE SITE

#### GENERAL

7.1 This section provides recommendations for assessing the potential for fault displacement phenomena that might challenge the safety of the nuclear installation. This challenge might arise from the potential to produce permanent offsetting or tearing of the ground surface (i.e. surface faulting) at or in the vicinity of the site for both new and existing nuclear installations affecting structures, systems and components important to safety. It also provides recommendations regarding the scope of the investigations that are necessary to permit such an assessment to be made.

7.2 In relation to this seismic hazard, SSR-1 [1] states that:

"Faults beyond a certain size and within a certain distance of the nuclear installation that are significant to safety shall be evaluated to identify whether these faults are to be considered capable faults. For capable faults, potential challenges to the safety of the nuclear installation in terms of ground motion and/or fault displacement hazard shall be evaluated." (Requirement 15 of SSR-1 [1])

"Capable faults shall be identified and evaluated. The evaluation shall consider the fault characteristics in the vicinity of the site. The methods used, and the investigations made shall be sufficiently detailed to support safety related decisions." (para. 5.2 of SSR-1 [1])

"The effect of fault displacement on safety related structures, systems and components shall be evaluated. Evaluation of fault displacement hazard shall include detailed geological mapping of excavations for safety related engineered structures to enable evaluation of fault capability at the site." (para. 5.3 of SSR-1 [1])

"For the new sites, an alternative site shall be considered when reliable evidence shows the existence of a capable fault and its effects cannot be compensated by design/engineering protective measures. In case of a capable fault is identified in the site vicinity of an existing nuclear installation, the site shall be deemed unsuitable if the site safety cannot be demonstrated." (para. 5.4 of SSR-1 [1])

The recommendations provided in this section are aimed at meeting these requirements, with special consideration given to the differences between new and existing sites.

7.3 Fault displacement is the relative movement of the two sides of a fault at or near the surface, measured in any chosen direction, in relation to an earthquake (either directly or indirectly). Principal faulting occurs along a main fault rupture plane (or planes) that is the location of release of the energy. Secondary or distributed faulting is the rupture that occurs near the principal faulting, possibly on splays of the main fault or antithetic faults. In other

words, displacements could be associated with the causative (i.e. seismogenic) fault or could occur co-seismically on secondary faults. It should be noted that tectonic relative displacements associated with folds (synclines and anticlines) are also included in the term 'fault displacement'. However, fault creep, when demonstrated as such, is considered as a slowly progressing geological hazard.

#### CAPABLE FAULTS

#### Definition

7.4 The first question regarding the assessment of the potential for fault displacement is whether a fault (buried or outcropping) at or in the vicinity of the site is to be considered as capable, i.e. whether or not a fault has a significant potential for producing displacement at or near the ground surface. The basis for answering such a question should be the proper analysis and interpretation of the data compiled in the integrated database (see Section 3), as incorporated in the seismic source model(s) (see Section 4), together with additional specific data that may be needed for such assessment.

7.5 Based on the geological, geophysical, geodetic and/or seismological data, a fault should be considered capable if the following conditions apply:

- a) If it shows evidence of past movement (such as significant deformations and/or dislocations) within such a period that it is reasonable to conclude that further movements at or near the surface might occur. In highly active areas, where both seismic and geological data consistently reveal short earthquake recurrence intervals, evidence of past movements in the period of Upper Pleistocene to Holocene (i.e. the present) may be appropriate for the assessment of capable faults. In less active areas, it is likely that much longer periods (e.g. Pliocene to Quaternary, i.e. the present) are appropriate. In areas where the observed activity is between these two rates (i.e. not as highly active as plate boundaries and not as stable as cratonic zones), one way to calibrate the time frame for fault capability may be to check if the site is in the deformed area of major regional faults. Longer time frames should be used when the site is far away from the potentially deformed areas of these regional structures.
- b) If the capability of a fault cannot be assessed as indicated above because it is not possible to obtain reliable geochronological data by any available method, the fault should be considered capable if it could be structurally linked with a known capable fault (i.e. if a structural relationship with a known capable fault has been demonstrated such that the movement of one fault may cause movement of the other at or near the surface).
- c) If the capability of a fault cannot be assessed as described in (a) and (b) because it is not possible to obtain the relevant reliable data by any available method, the fault should be considered capable if the maximum potential magnitude associated with

the seismogenic structure, as determined in Section 4, is sufficiently large and at such a depth (i.e. sufficiently shallow) that it is reasonable to conclude that, in the current tectonic setting of the site area, movement at or near the surface could occur.

7.6 The period, within which evidence of past movement will determine the capability of a fault as indicated in para. 7.5(a), should be defined at the beginning of the seismic hazard assessment project through a site specific criterion based on the characteristics of the regional tectonic environment and the conditions in the near region and site vicinity areas. This criterion for assessing fault capability should be established by or agreed with the regulatory body.

#### Investigations necessary to determine capability

7.7 Sufficient surface and subsurface related data should be obtained from the investigations in the regional, near regional, site vicinity and site areas (see Section 3) to show the absence of faulting at or near the site, or, if faults are present, to describe the direction, extent, history and rate of movements on these faults as well as the age of the most recent movement.

7.8 When surface faulting is known or suspected to be present, site vicinity scale investigations should be made that include very detailed geological and geomorphological mapping, topographical analyses, geophysical surveys (including geodetic measurements, if necessary), trenching, boreholes, age dating of sediments or faulted rock, local seismological investigations, and any other appropriate up to date and state-of-the-art techniques and remote sensing methods available, to ascertain the amount and age of previous displacements or deformations.

7.9 Consideration should be given to the possibility that faults that have not shown recent near surface movement might be reactivated by reservoir loading, fluid injection, fluid withdrawal or other such phenomena.

#### CAPABLE FAULT ISSUES FOR PROPOSED NEW SITES

7.10 If during the selection and evaluation stages of a proposed new site for a nuclear installation, reliable evidence is collected demonstrating the existence within the site vicinity and site areas of a capable fault with potential for seismogenic (i.e. primary) fault displacement phenomena, this issue should be treated as an exclusionary attribute (see para. 3.8 of IAEA Safety Standards Series No. SSG-35, Site Survey and Site Selection for Nuclear Installations [9]) and an alternative site should be considered.

7.11 If during the selection and evaluation stages of a proposed new site for a nuclear installation, reliable evidence is collected demonstrating the existence within the site vicinity area of a secondary fault belonging to a seismogenic capable fault located outside the site vicinity, this issue may be treated as a discretionary attribute (see para. 3.8 of SSG-35 [9]). However, if reliable evidence shows that this secondary fault is traced or extended to the site area, this issue should be treated also as an exclusionary attribute and an alternative site should be considered.

## CAPABLE FAULT ISSUES FOR SITES WITH EXISTING NUCLEAR INSTALLATIONS

7.12 In general, because of the extensive site investigation programme required for a nuclear installation, the situation should not arise in which further consideration should be given to the potential for fault displacement at the site of an existing nuclear installation. However, it may be the case that information comes to light later that there is a potential capable fault in the site vicinity that requires the assessment of fault displacement potential. Therefore, it is recommended that for existing nuclear installations for which a seismic safety evaluation programme is conducted (see IAEA Safety Standards Series No. NS-G-2.13, Evaluation of Seismic Safety for Existing Nuclear Installations [10]), the programme includes the assessment of the fault displacement potential based on the information available from the original site selection and evaluation stages, and then making use of updated information and current techniques and criteria, and using a proper interpretation for all newly available data.

7.13 If a new nuclear installation is to be built on a site on which there is already one or more existing nuclear installations, and information comes to light that there is a potential capable fault in its site vicinity, the approach for the new installation should be as recommended in paras 7.10 and 7.11.

7.14 If there is a potentially capable fault, either primary or secondary, within the site vicinity and site areas, it should first be determined whether the fault could potentially approach and subsequently cause surface displacement that affects items important to safety of the nuclear installation. This evaluation should be based on the characteristics of the fault, such as its sense of slip, geometry (length and width including strike dip and rake angles) and, for secondary faults, its structural relationship with the causative fault, and should use validated models (including dynamic rupture models) in a conservative way including due consideration of related uncertainties, both epistemic and aleatory.

7.15 If no sufficient basis is provided to decide conclusively that the fault is not capable, and the identified fault has a potential to affect the foundations of items important to safety of the nuclear installation, then, using all the available data compiled as recommended in Section 3, probabilistic methods should be used to obtain an estimate of the annual frequency of exceedance of various amounts of displacement at or near the surface.

7.16 In the probabilistic fault displacement hazard analysis, the following two types of possible displacements should be considered with careful and appropriate treatment of the involved uncertainties (both epistemic and aleatory):

- (a) Primary displacement, typically in the form of direct seismogenic fault rupture;
- (b) Secondary displacement (also called indirect or subsidiary displacement), typically associated with induced movement along pre-existing slip planes (e.g. a triggered slip on an existing fault or a bedding fault plane from an earthquake that occurred on another fault).

The fault displacement is generally characterized as a three-dimensional displacement vector that

should be resolved into components of slip along the fault trace and along the fault dip, with the resulting amplitude equal to the total evaluated slip (for a given annual frequency of exceedance and for a given fractile of hazard).

7.17 The annual frequency of exceedance corresponding to various amounts of displacement at or near the surface should be determined at the foundation points defined by the specific layout of foundations of structures, systems and components important to safety of the nuclear installation. The most up to date and reliable methods of probabilistic assessment should be applied. These include empirical relationships, and/or engineering models (such as finite element analysis or Coulomb static stress transfer models) that are compatible with the faulting type and site area specific geologic setting and using all available data.

7.18 The range of annual frequencies of exceedance, for which the amount of displacements is to be calculated, should be compatible with the safety principles of the nuclear installation. From the hazard curve thus obtained, the annual frequency of exceedance corresponding to the level required for safety evaluation purposes should be adopted to establish the corresponding surface rupture evaluation basis to conduct the safety evaluation of the installation.

## 8. PARAMETERS FROM THE VIBRATORY GROUND MOTION ANALYSIS, FAULT DISPLACEMENT AND OTHER ASSOCIATED SEISMIC HAZARDS

#### GENERAL

8.1 This section provides recommendations for establishing the parameters obtained from the analysis for evaluating, in general, the hazards generated by earthquakes at the site of the nuclear installation and which are aimed at providing the data necessary for defining the respective design or evaluation basis in accordance with the specific requirements for each type of installation.

#### SEISMIC VIBRATORY GROUND MOTION HAZARDS

#### Parameters and control point

8.2 Regardless of the method applied (i.e. a probabilistic approach or a deterministic approach, or both), the seismic vibratory ground motion hazards at the site should be defined by means of appropriate parameters such as spectral representations and time histories.

8.3 In principle, the vibratory ground motion parameters should be defined at the control point established by the user requirements (see Section 10). Usually, the control point is defined at free field conditions, i.e. at the ground surface or at key embedment depths, or at bedrock level. In cases where surface soil layers will be completely removed, the parameters should be defined at the level of the outcrop that will exist after removal. Consideration should be given to the appropriate interfacing of the defined reference ground motion and the site soil response analysis.

#### Site response analysis

8.4 The site response analysis performed as recommended in paras 6.16–6.21 provides the vibratory ground motion parameters at locations relevant for the design of the nuclear installation, e.g. at the free field ground surface, at foundation level or at any other level.

#### **Spectral representations**

8.5 The seismic vibratory ground motion hazard, calculated as recommended in Section 6, should be characterized by response spectra in horizontal and vertical components at the control point.

#### Uniform hazard response spectra

8.6 A uniform hazard response spectrum is developed by selecting the values of the response spectral ordinates that correspond to the annual frequencies of exceedance of interest from the seismic hazard curves. One or more uniform hazard response spectra may be developed from the results of the probabilistic seismic hazard analysis and any subsequent site response analyses that have been performed.

#### Response spectra based on scenario earthquakes

8.7 In deterministic seismic hazard assessments as well as after the de-aggregation process in the probabilistic seismic hazard assessments, scenario earthquakes should be used to realistically represent the frequency content of earthquakes. Scenario earthquakes resulting from the de-aggregation process for the results of probabilistic seismic hazard assessments should be associated with annual frequency of exceedance values.

#### Standardized response spectra

8.8 A standardized response spectrum having a smooth shape is used for engineering design purposes and to account for the contribution of multiple seismic sources represented by an envelope incorporating adequate low frequency and high frequency ground motion input. The prescribed shape of the standardized response spectrum is obtained from various response spectra based on earthquake records and engineering considerations. This standardized response spectrum is scaled to envelop the mean ground motion levels in a wide frequency range.

8.9 It is possible to have low to moderate magnitude near field earthquakes that have a relatively rich high frequency content and short duration with a high peak acceleration. The use of the peak acceleration from this type of earthquake to scale a broad banded standardized response spectrum could lead to an unrealistic shape for the standardized response spectra. In such a case, it is preferable to use multiple response spectra for design purposes to reflect properly the different types of seismic sources.

#### **Time histories**

8.10 Time histories should satisfactorily reflect all the prescribed ground motion parameters as embodied in the response spectra or other spectral representation with the addition of other parameters such as duration, phase and coherence. The number of time histories to be used in the detailed analyses and the procedure to be used in generating these time histories will depend on the type of analysis to be performed and should be specified by the user (see Section 10) on the basis of the different types of engineering analyses to be conducted during the design or evaluation stages.

8.11 Significant progress has been made in ground motion simulation based on fault rupture modelling, including fault rupture simulation, wave propagation paths and site effects (e.g. by use of empirical Green's function methods). Ground motions obtained in this way for regions for which pertinent parameters are available can be employed to complement the more traditional methods. Any time history should be applied carefully, especially when developed for soils that are expected to respond non-linearly.

8.12 In using response spectra to develop design time histories, it should be ensured that the time histories include the appropriate energy content represented by the design ground motions. This could be done by calculating the corresponding power spectral density functions.

#### **Ground motion duration**

8.13 The duration of an earthquake ground motion is determined by many factors, including the size of fault rupture (generally characterized by magnitude), crustal parameters along the propagation path (generally characterized by distance), and conditions beneath the site such as the presence of a significant sedimentary column. A consistent definition of duration should be used throughout the evaluation. Common definitions of duration include:

- (a) The time interval between the onset of ground motion and the time at which the acceleration has declined to 5% of its peak value;
- (b) The time interval between the 95<sup>th</sup> and 5<sup>th</sup> percentiles of the integral of the mean square value of the acceleration;
- (c) The time interval for which the acceleration exceeds 5% of the acceleration due to gravity, g.

8.14 In determining an appropriate duration for the time histories, due weighting should be given to any empirical evidence provided by the regional database. For some sites, relatively low amplitude motions from distant, large earthquakes might pose a liquefaction hazard. When this condition applies, time histories used for liquefaction should include such low amplitude time histories over an appropriate duration.

#### Vertical ground motion

8.15 Vertical vibratory ground motions (response spectra and time histories) should be developed by using the same methods as are used for developing horizontal vibratory ground motions. However, if vertical attenuation relationships are not available, it may be reasonable to assume a ratio between vertical and horizontal ground motion that is prescribed by current best practice.

#### Ground motion for base isolated structures, buried structures and fuel pools

8.16 The methodology for deriving the design ground motions has been developed for installation structures having conventional foundations. For structures that utilize base isolation systems for protection of the installation against earthquake generated vibratory ground motions, additional considerations may be necessary, including the careful review of worldwide experience in relation to approved specific performance and design criteria as well as corresponding regulatory requirements. Of most concern are long period effects that might cause excessive residual displacements in the elements of the base isolation system. For plant structures for which a base isolation system is envisaged, time histories should be examined and, if necessary, modified to take these long period (and potentially long duration) effects into account. The evaluation should consider surface wave influences due to thick sediments.

8.17 For buried structures such as ducts and piping, appropriate response spectra and time histories should be developed in cooperation with the structural designer.

8.18 Appropriate vibratory ground motion representation should be developed when the project plan calls for the consideration of sloshing effects in pools or ponds.

#### FAULT DISPLACEMENT

8.19 For existing nuclear installations for which a fault displacement hazard analysis was performed in accordance with paras 7.12–7.18, the surface fault displacement associated with each capable fault that can produce surface faulting in the site area should be determined from the values of surface fault displacement hazards. These values should correspond to the acceptable value of the annual frequencies of exceedance specified in accordance with the safety requirements established in SSR-1 [1], and as specified in the project plan. The empirical fault displacement models have a larger uncertainty than vibratory ground motion models due to there being less data available and this should be taken into consideration.

#### EVALUATION OF OTHER HAZARDS ASSOCIATED WITH EARTHQUAKES

8.20 Aside from the evaluation of the ground motion and surface faulting hazards, the results of a seismic hazard analysis should be used in the assessment of other hazards associated with earthquakes that might be significant for the safety of nuclear installations. These hazards include tsunamis, liquefaction, slope instability, subsidence, subsurface cavities, karstic processes and the failure of water retaining structures, which might be initiated either by ground motion or by surface faulting. A thorough assessment should be carried out to determine the level of seismic hazard or the supporting models appropriate for the associated hazard under consideration.

#### Tsunamis

8.21 For coastal sites, the potential for tsunamis should be carefully evaluated in the framework of hydrological hazards (see SSG-18 [4]). Tsunamis can be generated by earthquakes that cause tectonic deformation of the seabed or submarine landslides. For tectonically generated tsunamis, the region of investigation may be very large, extending to several thousands of kilometers in radius. In such cases, the part of the tsunamigenic sources that are included in the region relevant to the seismic hazard evaluation (generally considered to be about 300 km in radius) should be consistent with the seismogenic attributes of these faults, while taking into account that faults on land and those that do not generate significant vertical motion would not contribute significantly to the tsunami hazard.

8.22 For a tsunami hazard associated with near regional submarine landslides, the seismic hazard appropriate for triggering the landslide should be determined consistently with the hazard level associated with the nuclear installation.

8.23 For evaluating the fault related tsunami hazards, the coastal subsidence should be estimated. A paleotsunami study should be conducted within the near region to understand the history of tsunamis on the coast. These studies may be part of the seismic hazard study or the tsunami hazard study but in any case, the work should be coordinated.

#### Liquefaction potential

8.24 Non-cohesive soils in loosely deposited conditions below the water table are susceptible

to liquefaction; if this is the case, the bearing capacity of the soil decreases significantly when subjected to vibratory ground motions. Therefore, careful geotechnical investigations should be carried out in the site area to assess the liquefaction potential that might affect the safety of the systems, structures and components of the nuclear installation.

8.25 For soils susceptible to liquefaction, detailed information on the design soil profile is needed and it should be obtained as described in paras 3.16 and 3.17 of NS-G-3.6 [3]. For assessing the liquefaction potential using any of the three methods described in paras 3.18–3.25 of NS-G-3.6 [3], the specific characteristics of the earthquake design basis, or seismic hazard at the site, should be provided accordingly. Therefore, the earthquake magnitude values for different design conditions should be properly defined using the corresponding information and data used for the seismic hazard analysis, in case other empirical approaches are used for assessing the liquefaction potential (see para 3.19 of NS-G-3.6 [3]). The same approach should be followed in relation to the appropriate selection of the time histories to define the number of cycles of stress and the adequate input motions for non-linear stress analysis, as needed for analytical approaches (see paras 3.20–3.25 of NS-G-3.6 [3]). In any case, close coordination should be established with the geotechnical engineering team performing the liquefaction analysis and foundation design.

#### **Slope stability**

8.26 The stability of natural and human-built slopes located in the site area that can be affected by the vibratory ground motions should be investigated since landslides could seriously affect structures, systems and components important to safety. The evaluation of the stability of slopes should be done using appropriate parameters of the vibratory ground motions obtained from the seismic hazard analysis at the site. As described in para. 5.5 of NS-G-3.6 [3], the peak ground acceleration of the seismic design basis is usually the parameter used for estimating the inertial loads although in some cases a more refined dynamic analysis may be necessary.

#### Collapse due to cavities and subsidence phenomena

8.27 The potential for complex subsurface conditions should be investigated, as recommended in paragraphs 2.35–2.47 of NS-G-3.6 [3]. Such conditions at the site area could have serious implications for the integrity of the foundation of items important to safety of the nuclear installation. When performing the seismic hazard assessment for a nuclear installation site, the prediction, detection and evaluation should proceed using data and methods adequate for such purposes. As cavities can preferentially develop along fault lines, the potential for co-seismic movement on these should be investigated.

#### Failure of water retaining structures (dam break)

8.28 The failure of water retaining structures located upstream of the site area due to a seismic event should be investigated considering the consequential flooding hazards that might affect the safety of the nuclear installation. Therefore, the earthquake design basis, including the seismic hazard and the performance and safety criteria, adopted for such structures should be obtained from the authorities and organizations responsible for such structures. This

information should be properly analysed, including the specific characteristics (e.g. water mass of the dams), to ensure the safety of the nuclear installation at the site or to implement adequate site related mitigation measures.

8.29 Consideration should be given to the possible existence of several dams in the upper stream region, for which domino effects could occur. Hydrological energy should be considered based on the inundation level as well as the velocity of the water flow. A landslide might produce natural dams and the potential for these dams to break is highly uncertainty.

8.30 If all the seismogenic sources that might affect the water retaining structure(s) to be considered are within the region of investigation for the seismic hazard analysis of the nuclear installation, then the same seismic source characterization and ground motion characterization models should be used in the seismic hazard assessment of these water retaining structure(s). If this is not the case, seismic sources common to both the nuclear installation and the water retaining structure(s) should be modelled taking into consideration the attributes used in the seismic hazard analysis of the nuclear installation. In any case, close coordination should be established with the hydrological engineering team performing the dam break analysis and protection against flooding.

#### Volcano related phenomena

8.31 Earthquakes and related hazards are phenomena associated with volcanic events, as indicated in Table 1 of IAEA Safety Standards Series No. SSG-21, Volcanic Hazards in Site Evaluation for Nuclear Installations [11]. Earthquakes generated by volcanic activity are typically smaller than tectonic earthquakes. In the case that a (seismogenic) capable fault is identified in the vicinity of an active volcano, both seismic and volcanic hazard should be taken into consideration since earthquakes might occur on the capable fault preceding, accompanying, or following the volcanic eruption as a result of the mutual influence of tectonic movement and magma intrusion. Also, the identification of aligned volcanic vents in a well-defined local area might indicate the presence of a tectonic fault, or possibly a capable fault.

# 9. EVALUATION OF SEISMIC HAZARDS FOR NUCLEAR INSTALLATIONS OTHER THAN NUCLEAR POWER PLANTS

#### GENERAL

9.1 In consideration of the use of the graded approach described in para. 1.8, this section provides guidance on seismic hazard evaluation for a broad range of nuclear installations (see para. 1.8) other than nuclear power plants.

9.2 The evaluation of the seismic hazards at nuclear installations other than nuclear power plants should be commensurate with the complexity of such installations, with the potential radiological hazards, and with hazards due to other materials present on the site

9.3 The recommended method for applying the graded approach is to start with attributes relating to nuclear power plants and, if possible, to commensurately adjust these for installations with which lesser radiological consequences are associated. If this approach is not practicable, the recommendations relating to nuclear power plants should be applied to other types of nuclear installation.

#### SCREENING PROCESS

9.4 Prior to adopting a graded approach, a conservative screening process should be applied in which it is assumed that the entire radioactive inventory of the installation is released by the potential seismically initiated accident. If the potential result of such a radioactive release is that unacceptable consequences would not be likely — for workers or the public (i.e. doses to workers and the public would be below the dose limits established by the regulatory body) or for the environment — and provided that no other specific requirements are imposed by the regulatory body for such an installation, the installation may be excluded from the requirement to undertake a full seismic hazard evaluation. If, even after such screening, some degree of seismic hazard evaluation is considered necessary, national seismic codes for hazardous and/or industrial facilities should be used.

9.5 The conservative screening process in para. 9.4 should be conducted considering the likelihood that a seismic event will result in an event with radiological consequences. This likelihood will highly depend on the following factors related to the characteristics of the nuclear installation (e.g. its purpose, layout, design, construction and operation):

- (a) The amount, type and status of the radioactive inventory at the site (e.g. whether solid, liquid and/or gaseous, and whether the radioactive material is being processed or only stored).
- (b) The intrinsic hazard associated with the physical processes (e.g. nuclear chain reactions) and chemical processes (e.g. for fuel processing purposes) that take place at the installation.
- (c) The thermal power of the nuclear installation, if applicable.
- (d) The configuration of the installation for different kinds of activity.

- (e) The distribution of radioactive sources in the installation (e.g. for research reactors, most of the radioactive inventory will be in the reactor core and the fuel storage pool, whereas for fuel processing and storage facilities it might be distributed throughout the installation).
- (f) The changing nature of the configuration and layout of installations designed for experiments (such activities have an associated intrinsic unpredictability).
- (g) The need for active safety systems and/or operator actions for the prevention of accidents and for mitigation of the consequences of accidents, and the characteristics of engineered safety features for the prevention of accidents and for mitigation of the consequences of accidents (e.g. the containment and containment systems).
- (h) The characteristics of the structures of the nuclear installations and the means of confinement of radioactive material.
- (i) The characteristics of the processes or of the engineering features that might show a cliff edge effect<sup>3</sup> in the event of an accident.
- (j) The characteristics of the site that are relevant to the consequences of the dispersion of radioactive material to the atmosphere and the hydrosphere (e.g. size, demographics of the region).
- (k) The potential for on-site and off-site contamination.

9.6 Depending on the criteria applied by the regulatory body, some or all the factors in para. 9.5 should be considered when applying the conservative screening process. For example, fuel damage, radioactive releases or doses may be the conditions or metrics that warrant special consideration.

9.7 If the results of the conservative screening process show that the potential consequences of such releases would be unacceptable, a seismic hazard evaluation of the installation should be carried out, starting by applying the recommendations relevant to nuclear power plants.

- 9.8 The application of the graded approach should be based on the following information:
  - (a) The existing safety analysis report for the installation, which should be the primary source of information.
  - (b) The results of a probabilistic safety assessment, if one has been performed.
  - (c) The characteristics specified in para. 9.5 above.

#### CATEGORIZATION PROCESS

9.9 If the conservative screening process (see para. 9.7), indicates that a seismic hazard evaluation of the installation is to be carried out, a process for categorizing installations should

 $<sup>^{3}</sup>$  A 'cliff edge effect' in a nuclear installation is an instance of severely abnormal system behaviour caused by an abrupt transition from one system status to another following a small deviation in a system parameter; and, thus, a sudden large variation in system conditions in response to a small variation in an input.

be undertaken to apply the graded approach. This categorization may be performed at the design stage or later. If the categorization has been performed, the assumptions on which it was based should be reviewed and verified. In general, the criteria for categorization should be based on the radiological consequences of radioactive releases from the installation, ranging from very low to potentially severe consequences. As an alternative, the categorization may consider the radiological consequences within the installation itself, the radiological consequences for the installation, and the radiological consequences for the public and the environment outside the site.

9.10 Three or more categories may be defined based on national practice and criteria, and on the information described in para. 9.8. As an example, the following categories may be defined:

- (a) The lowest hazard category includes those nuclear installations for which national building codes for conventional facilities (e.g. essential facilities such as hospitals) or for hazardous facilities (e.g. petrochemical or chemical plants) should be applied as a minimum.
- (b) The highest hazard category includes installations for which standards and codes for nuclear power plants should be applied.
- (c) There is often, at least, one intermediate category between (a) and (b) above, corresponding to a hazardous installation for which, as a minimum, codes dedicated to hazardous facilities should be applied.

#### VIBRATORY GROUND MOTION AND OTHERS HAZARD ANALYSIS

#### Vibratory ground motion hazard assessment

9.11 The vibratory ground motion seismic hazard assessment for the installations categorized as recommended in paras. 9.9 and 9.10, should be performed in accordance with the following:

- (a) For the least hazardous installations, the seismic hazard input for the design may be taken from national building codes and maps.
- (b) For installations in the highest hazard category, methodologies for seismic hazard assessment as described in Sections 3–8 of this Safety Guide (i.e. recommendations applicable to nuclear power plants) should be used.
- (c) For installations categorized in the intermediate hazard category, the following approach might be applicable:
  - (i) If the seismic hazard assessment is typically performed using methods similar to those described in this Safety Guide, a lower seismic input for designing these installations may be adopted at the design stage, in accordance with the safety requirements for the installation;
  - (ii) If the database and the methods recommended in this Safety Guide are found to be disproportionately complex, time consuming and demanding in terms of the nuclear installation in question, simplified methods for seismic hazard assessment

(that are based on a more restricted data set) may be used. In such cases, the seismic input finally adopted for designing these installations should be commensurate with the reduced database and the simplification of the methods, with account being taken of the fact that both factors tend to increase uncertainties.

9.12 The design basis ground motion levels for these nuclear installations should be decided in the context of the approach to hazard assessment recommended in para. 9.11.

9.13 The recommendations relating to seismic instrumentation installed on the site (see paras 3.51-3.56) should be applied in a manner that is commensurate with the category of the installation as defined in para. 9.10.

#### Associated geological and geotechnical seismic hazards

9.14 With regard to the associated geological and geotechnical seismic hazards, the same considerations as for nuclear power plants should apply. If there is reliable evidence that demonstrates that fault displacement phenomena could occur within the site vicinity and site areas that are associated with co-seismically secondary fault displacements, a detailed and specific fault displacement assessment should be conducted. The site may still be considered as suitable on the basis of specific established suitability criteria and, therefore, the corresponding surface faulting design basis should be established to ensure the safety of the nuclear installation through design, construction and operation measures.

#### 10. PROJECT MANAGEMENT SYSTEM

#### GENERAL ASPECTS OF PROJECT ORGANIZATION

10.1 Requirements for management for safety are established in IAEA Safety Standards Series No. GSR Part 2, Leadership and Management for Safety [12]. Further recommendations on the management system are provided in IAEA Safety Standards Series No. GS-G-3.1, Application of the Management System for Facilities and Activities [13].

10.2 Criteria for implementing a formal management system programme should be established by the study sponsor to be applied by the organization or consultancy firm in charge of conducting the seismic hazard assessment project. The sponsor should also identify the quality assurance standards to be met. Special provisions should be specified for document control, analysis control, software, validation and verification, procurement and audits, and non-conformance control and corrective actions. Work related documents should be prepared to cover all the activities for data collection and data processing, field and laboratory investigations, analyses and evaluations, and for expert related processes, such as procedures for experts' interaction that are within the scope of this Safety Guide.

10.3 As part of the overall project management system to be developed and maintained by the organization or consultancy firm conducting the seismic hazard assessment, a project quality manual should be prepared that, as a minimum, addresses the following topics:

- (a) The objectives and scope of the project;
- (b) Applicable regulations and standards;
- (c) Responsibilities for management of the project;
- (d) Description of processes and tasks;
- (e) Interfaces among the different types of tasks (e.g. field tasks, laboratory tests, analysis) and disciplines involved (e.g. earth sciences, engineering) with all inputoutput requirements;
- (f) Project deliverables and reporting.

10.4 The project scope should identify all the hazards generated by earthquakes that are relevant for the safety of the nuclear installation and that will be investigated within the framework of the project. This Safety Guide addresses a group of hazards that are associated with earthquakes. Depending on the objectives of the project, some or all of these hazards may be considered in the scope. If some of the hazards are considered to be out of scope because it is believed that they are not relevant to the site, a screening process should be applied to demonstrate and document that this is the case.

10.5 All regulatory documents and standards that apply to the project should be identified by the study sponsor and made available to the organization or consultancy firm conducting the project prior to the start of work, including any additional requirements and/or specific criteria imposed on the project, for example, in relation to the regulatory requirements of the supplier country of the nuclear installation.

10.6 All approaches and methodologies that reference lower tier legislation (e.g. regulatory guidance documents, industry codes and standards) should be clearly identified and described. If procedures for experts interaction are used to better capture epistemic uncertainties, the sophistication and complexity of these approaches should be chosen by the study sponsor based on the project requirements. The details of the approaches and methodologies to be used should be clearly stated in the project quality manual. These details should include the functions of different categories of experts (e.g. proponent, resource expert, technical integrator, review panel member), and their responsibilities with regard to the project management.

#### SPECIFIC ASPECTS OF PROJECT ORGANIZATION

10.7 A project work plan should be prepared prior to, and as a basis for, the execution of the seismic hazard assessment project. This work plan should include a description of all requirements that are relevant for the project, including applicable regulatory requirements in relation to all the hazards considered to be within the project scope. It is recommended that this set of requirements be reviewed by the regulatory body prior to conducting the seismic hazard analysis. In addition to these requirements, the project work plan should delineate the following specific elements: personnel and their responsibilities; work breakdown and project tasks; schedule and milestones; and the agreed deliverables, outputs and reports.

10.8 The project work plan should ensure that there is adequate provision, in the budget and in the schedule, for collecting new data that may be important for the conduct of the seismic hazard assessment and/or for responding to requests by experts, including provision for balancing potentially conflicting project needs.

10.9 The results of the seismic hazard analysis should include all outputs indicated in the project work plan. The Annex in this Safety Guide identifies typical results to be reported in all applications as well as others that may be needed by the study sponsor. The reporting of the seismic hazard analysis should be specified in sufficient detail in the project work plan.

10.10 To make the evaluation traceable and transparent to users (e.g. peer reviewers, the operating organization, the regulatory body, the designers, the vendors, the contractors and the subcontractors of the operating organization), the documentation for the seismic hazard analysis should provide a description of all elements of the seismic hazard analysis process and include the following information:

(a) Description of the study participants and their roles;

- (b) Background material that comprises the analysis documentation, including raw and processed data;
- (c) A description of the computer software used, and input and output files;
- (d) Reference documents;
- (e) All documents supporting the treatment of uncertainties, expert opinion and related discussions;
- (f) Results of intermediate calculations and sensitivity studies;
- (g) Final report(s).

10.11 This material should be maintained in an accessible, usable and auditable form by the study sponsor, for example using a geographical information system. Documentation or references that are readily available elsewhere should be cited where appropriate.

10.12 The documentation should identify all sources of information used in the seismic hazard analysis, including information on where to find important citations that might be difficult to obtain. Unpublished data that are used in the analysis should be included in the documentation in an appropriately accessible and usable form.

10.13 The documentation for the seismic hazard analysis should identify the computer software that was used. This should include computer programs used in the processing of data (e.g. the earthquake catalogue) and the computer programs used to perform calculations for the seismic hazard analysis.

10.14 Owing to the variety of investigations carried out (in the field, in the laboratory and in the office) and the need for expert judgement in the decision-making process, technical procedures that are specific to the project should be developed to guide and facilitate the execution and verification of these tasks.

#### ENGINEERING USES AND OUTPUT SPECIFICATION

10.15 A seismic hazard assessment is usually conducted for the purposes of seismic design and/or seismic probabilistic safety assessment of the nuclear installation. Therefore, from the beginning, the work plan for the seismic hazard assessment should identify the intended engineering uses and objectives of the assessment and should incorporate an output specification that describes all the results necessary for the intended engineering uses and objectives of the study.

10.16 To the extent possible, the output specification for the seismic hazard analysis should be comprehensive. The output specification may be updated, as necessary, to accommodate additional results, and/or to reduce the scope of the results. Elements that should be considered in the output specification include the following:

— *Ground motion parameters*. Specified ground motion parameters should be sufficient to produce the necessary results and any additional outputs needed for engineering

use (see the Annex for typical outputs of a probabilistic seismic hazard analysis for assessing the vibratory ground motion parameters).

- *Vibration frequencies.* In addition to specific client requirements, the range and density of specified vibration frequencies for the uniform hazard response spectra should be sufficient to adequately represent the input for all safety relevant structures, systems and components.
- *Damping*. Specified damping values should be sufficient to adequately represent input for, and the effects on, the responses of all safety relevant structures, systems and components.
- *Ground motion components*. The output of both vertical and horizontal motions should be specified.
- *The reference subsurface rock site condition.* For studies where site response analysis is performed, the output specification should include a definition of the rock conditions on the site (usually for a depth significantly greater than 30 metres, corresponding to a specified value of the shear wave velocity consistent with firm rock). The analysis results should correspond to this reference condition.
- Control point(s). The output specification should specify the control points (e.g. depths at the site) for which near surface vibratory ground motion hazard results are obtained. Usually, the control points include the ground surface and key embedment depths (e.g. foundation levels) for structures and components. The specified control points should be sufficient to develop adequate input(s) for soil–structure interaction analyses.

10.17 In any seismic hazard assessment, there is a need to consider a lower bound magnitude owing to constraints in the seismological database. Therefore, in addition to the specification of outputs for anticipated engineering uses, the project plan should specify the following additional parameters relating to engineering validity and/or the utility of the seismic hazard analysis:

- Lower bound motion filter. Use of a lower bound motion is needed to develop a practical computation for seismic hazard analysis, and the lower bound motion should be selected to include all potentially damaging and risk significant events. The lower bound motion filter should be selected in consultation with the seismic designer and/or the fragility analyst for the seismic probabilistic safety assessment, who should agree that the filter is set to capture all potentially damaging or risk significant events.
- *Lower bound magnitude*. In addition to previous recommendations, a selected lower bound magnitude should not exceed  $M_w = 5.0$ .
- As an alternative to the use of a magnitude measure such as M<sub>w</sub>, the lower bound motion filter may be specified in terms of an established damage parameter, such as cumulative absolute velocity, in conjunction with a specific value of that parameter

for which it can be clearly demonstrated that no contribution to damage or risk will occur.

#### INDEPENDENT PEER REVIEW

10.18 In view of the complexity of the seismic hazard assessment, an independent peer review procedure should be proposed and implemented as part of the project work plan and be conducted to provide assurance that: (i) a proper process has been duly followed in conducting the seismic hazard analysis, (ii) the analysis has addressed and evaluated the involved uncertainties (both, epistemic and aleatory), and (iii) that the documentation is complete and traceable.

10.19 Two methods of peer review can be used: participatory peer review, and late stage peer review. A participatory peer review is carried out during the study, allowing the reviewer(s) to resolve comments as the seismic hazard analysis proceeds and as technical issues arise. A late stage and follow-up peer review is carried out towards the end of the study. Participatory peer review will decrease the likelihood of the study being rejected at a late stage.

10.20 An independent peer review procedure should be conducted to address all parts of the seismic hazard assessment, including the compilation and evaluation of the available data, the process for the seismic hazard analysis, all technical elements (e.g. seismic source characterization, ground motion evaluation), the method of seismic hazard analysis, and quantification and documentation. The procedure should be based on the participation of duly qualified multidisciplinary team of experts and the principle of integration of their different professional judgements. The procedure should include the conduct of technical meetings or workshops for discussing the reliability and quality of available data, the safety significance of hazards issues, the alternative interpretations, and for providing feedback to the project team. The number and timing for these workshops will be established in the proposed work plan in accordance with the necessary level of effort and the available resources and they should be duly documented and reported.

10.21 The independent peer review team members should include the multidisciplinary expertise to address all technical and process related aspects of the analysis. The peer reviewer(s) should not have been involved in other aspects of the seismic hazard analysis and should not have a vested interest in the outcome. The level and type of peer review can vary, depending on the application of the seismic hazard analysis.

10.22 In dealing with seismic source characterization issues, it may be possible for project experts to recognize and represent the centre, the body and the range of technically defensible interpretations through interactions with experts not directly involved with the project ('invited experts') who participate to provide their specific interpretation and professional judgement on the subject or issue under discussion. It is recommended that invited experts provide their input to the independent peer review team, although they will not be responsible for this part of the process. This approach is most suitable for topics that pertain to regional modelling issues, while it should be recognized that for near region and the site vicinity scales,

invited experts might not adequately provide diversity because they do not possess project specific data. It is the diversity of the expertise within the project that will make it possible to address the appropriate representation of epistemic uncertainties.

#### REFERENCES

- [1] INTERNATIONAL ATOMIC ENERGY AGENCY, Site Evaluation for Nuclear Installations, IAEA Safety Standards Series No. SSR-1, IAEA, Vienna (in preparation).
- [2] INTERNATIONAL ATOMIC ENERGY AGENCY, IAEA Safety Glossary: Terminology Used in Nuclear Safety and Radiation Protection, 2016 Edition, IAEA, Vienna (2016).
- [3] INTERNATIONAL ATOMIC ENERGY AGENCY, Geotechnical Aspects of Site Evaluation and Foundations for Nuclear Power Plants, IAEA Safety Standards Series No. NS-G-3.6, IAEA, Vienna (2004).
- [4] INTERNATIONAL ATOMIC ENERGY AGENCY, Meteorological and Hydrological Hazards in Site Evaluation for Nuclear Installations, IAEA Safety Standards Series No. SSG-18, IAEA, Vienna (2011).
- [5] INTERNATIONAL ATOMIC ENERGY AGENCY, Seismic Design and Qualification for Nuclear Power Plants, IAEA Safety Standards Series No. NS-G-1.6, IAEA, Vienna (2003). (revision in preparation).
- [6] U.S. Nuclear Regulatory Commission (NRC), Practical Implementation Guidelines for SSHAC Level 3 and 4 Hazard Studies, NUREG-2117 (Rev. 1), NRC, Washington, DC (2012).
- [7] INTERNATIONAL ATOMIC ENERGY AGENCY, Safety of Nuclear Power Plants: Design, Specific Safety Requirements, IAEA Safety Standards Series No. SSR-2/1 (Rev. 1), IAEA, Vienna (2016).
- [8] INTERNATIONAL ATOMIC ENERGY AGENCY, Development and Application of Level 1 Probabilistic Safety Assessment for Nuclear Power Plants, IAEA Safety Standards Series No. SSG-3, IAEA, Vienna (2010).
- [9] INTERNATIONAL ATOMIC ENERGY AGENCY, Site Survey and Site Selection for Nuclear Installations, IAEA Safety Standards Series No. SSG-35, IAEA, Vienna (2015)
- [10] INTERNATIONAL ATOMIC ENERGY AGENCY, Evaluation of Seismic Safety for Existing Nuclear Installations, IAEA Safety Standards Series No. NS-G-2.13, IAEA, Vienna (2009).
- [11] INTERNATIONAL ATOMIC ENERGY AGENCY, Volcanic Hazards in Site Evaluation for Nuclear Installations, IAEA Safety Standards Series No. SSG-21, IAEA, Vienna (2012).
- [12] INTERNATIONAL ATOMIC ENERGY AGENCY, Leadership and Management for Safety, IAEA Safety Standards Series No. GSR Part 2, , IAEA, Vienna (2016).
- [13] INTERNATIONAL ATOMIC ENERGY AGENCY, Application of the Management

System for Facilities and Activities, IAEA Safety Standards Series No. GS-G-3.1, IAEA, Vienna (2006).

## ANNEX-TYPICAL OUTPUT OF PROBABILISTIC SEISMIC HAZARD ANALYSES

## TABLE A–1. TYPICAL OUTPUT OF PROBABILISTIC SEISMIC HAZARD ANALYSES

Output	Description	Format
Mean hazard curves	Mean annual frequency of exceedance for each ground motion level of interest associated with the suite of epistemic hazard curves generated in the probabilistic seismic hazard analysis.	Mean hazard curves should be reported for each ground motion parameter of interest in tabular as well as graphic format.
Fractile hazard curves	Fractile annual frequency of exceedance for each ground motion level of interest associated with the suite of epistemic hazard curves generated in the probabilistic seismic hazard analysis.	Fractile hazard curves should be reported for each ground motion parameter of interest in tabular as well as graphic format. Unless otherwise specified in the work plan, fractile levels of 0.05, 0.16, 0.50, 0.84 and 0.95 should be reported.
Uniform hazard response spectra	Response spectra whose ordinates have an equal probability of being exceeded, as derived from seismic hazard curves.	Mean and fractile uniform hazard response spectra should be reported in tabular as well as graphic format. Unless otherwise specified in the work plan, the uniform hazard response spectra should be reported for annual

frequencies of exceedance of  $10^{-2}$ ,  $10^{-3}$ ,  $10^{-4}$ ,  $10^{-5}$  and  $10^{-6}$  and for fractile levels of 0.05, 0.16, 0.50, 0.84 and 0.95.

 Magnitude–
 A magnitude–distance (M–D)

 distance
 deaggregation quantifies the

 deaggregation
 relative contribution to the total

 mean hazard of earthquakes that
 occur in specified magnitude–

 distance ranges (i.e. bins).

Mean andThe M–D deaggregation resultsmodalprovide the relative contributionmagnitudeto the site hazard of earthquakesand distanceof different sizes and at differentdistances. From thesedistributions, the mean and/ormodal magnitudes and the meanand/or modal distances ofearthquakes that contribute to thehazard can be determined.

SeismicThe seismic hazard at a site is asourcecombination of the hazard fromdeaggregationindividual seismic sources

The M–D deaggregation should be presented for ground motion levels corresponding to selected annual frequencies of exceedance for each ground motion parameter considered in the probabilistic seismic hazard analysis. The deaggregation should be performed for the mean hazard and for the annual frequencies of exceedance to be used in the evaluation or design.

The mean and modal magnitudes and distances should be reported for each ground motion parameter and level for which the M–D deaggregated hazard results are given. Unless otherwise specified in the work plan, these results should be reported for response spectral frequencies of 1, 2.5, 5 and 10 Hz.

The seismic source deaggregation should be reported for ground motion levels

	modelled in the probabilistic	corresponding to each ground
	seismic hazard analysis. A	motion parameter considered in
	deaggregation on the basis of	the probabilistic seismic hazard
	seismic sources provides an	analysis. The deaggregation
	insight into the possible location	should be performed for the
	and type of future earthquake	mean hazard and presented as a
	occurrences.	series of seismic hazard curves
d	In a probabilistic seismic hazard	A group of aggregated discrete
ves	analysis, often thousands to	hazard curves, each with an

Aggregated group of aggregated discrete hazard curv zard curves, each with an millions of hazard curves are assigned probability weight, generated to account for should be reported in tabular as epistemic uncertainty. For use in well as graphic format. certain applications (e.g. a seismic probabilistic safety assessment), a smaller, more manageable set of curves is required. Aggregation methods are used to combine like curves that preserve the diversity in shape of the original curves as well as the essential properties of the original set (e.g. the mean hazard). Earthquake For the purposes of engineering The format for presenting time histories analysis, time histories may be earthquake time histories will generally be defined in the work required that are consistent with the results of the probabilistic plan.

seismic hazard analysis. The criteria for selecting and/or

generating a time history may be

specified in the work plan. Example criteria include the selection of time histories that are consistent with the mean and modal magnitudes and distances for a specified ground motion or annual frequency of exceedance.

## DEFINITIONS

- **accelerogram.** A recording of ground acceleration, usually in three orthogonal directions (i.e. components), two in the horizontal plane and one in the vertical plane.
- **aleatory uncertainty.** Uncertainty inherent in a phenomenon. Aleatory uncertainty is taken into account by representing the phenomenon in terms of a probability distribution model.
- **craton.** A part of the Earth's crust that has attained stability, and has been little deformed for a prolonged period. The term is used to distinguish the stable portion of the continental crust from regions that are more geologically active and unstable. Cratons are generally found in the interiors of tectonic plates. They are characteristically composed of ancient crystalline basement rock, which may be covered by younger sedimentary rock.
- epicentre. The point on the Earth's surface directly above the focus (i.e. hypocentre) of an earthquake.
- **epistemic uncertainty.** Uncertainty attributable to incomplete knowledge about a phenomenon, which affects the ability to model it. Epistemic uncertainty is reflected in a range of viable models, multiple expert interpretations and statistical confidence.
- **fault (geological).** A planar or gently curved fracture surface or zone of the Earth across which there has been relative displacement.
- **free field ground motion.** Motion that would occur at a given point on the ground owing to an earthquake if vibratory characteristics were not affected by structures and facilities.
- frequency of exceedance. The frequency at which a specified level

of seismic hazard will be exceeded at a site or in a region within a specified time interval. In probabilistic seismic hazard analysis (PSHA), generally a one year time interval (i.e. annual frequency) is assumed. When the frequency is very small and it cannot exceed unity (in the prescribed interval), this number approaches the probability of the same event when the random process is assumed to be Poissonian.

- **hypocentre.** The point (focus) within the Earth at which an earthquake is initiated.
- **interplate.** Of tectonic processes, at the interfaces between the Earth's tectonic plates.
- intraplate. Of tectonic processes, within the Earth's tectonic plates.
- **magnitude** (of an earthquake). Measure of the size of an earthquake relating to the energy released in the form of seismic waves. Seismic magnitude means the numerical value on a standardized scale such as, but not limited to, moment magnitude, surface wave magnitude, body wave magnitude, local magnitude or duration magnitude.
- **maximum potential magnitude.** Reference value used in seismic hazard analysis characterizing the potential of a seismic source to generate earthquakes. The way in which it is calculated depends on the type of seismic source considered and the approach to be used in the seismic hazard analysis.
- **neotectonics.** The study of the motions and deformations of Earth's crust (geological and geomorphological processes) that are current or recent in geologic time.
- **palaeoseismicity.** The evidence of a prehistoric or historical earthquake manifested as displacement on a fault or secondary effects such as ground deformation (i.e. liquefaction, tsunami, landslides).

- **peak ground acceleration.** The maximum absolute value of ground acceleration displayed on an accelerogram; the greatest ground acceleration produced by an earthquake at a site.
- **response spectrum.** A curve calculated from an accelerogram that gives the value of peak response in terms of the acceleration, velocity or displacement of a damped single-degree-of-freedom linear oscillator (with a given damping ratio) as a function of its natural frequency or period of vibration.
- **seismogenic structure.** A structure that displays earthquake activity or that manifests historical surface rupture or the effects of palaeoseismicity, and that is considered likely to generate macro-earthquakes within a time period of concern.
- **seismic source model.** The model that defines the characterization of seismic sources in the region around a site of interest, including the aleatory and epistemic uncertainties in the seismic source characteristics.
- **site response.** The behaviour of a rock or soil column at a site under a prescribed ground motion load.
- **spectral acceleration.** Peak acceleration response of a linear onedegree of freedom oscillator as a function of its natural period or frequency and damping ratio when subjected to an acceleration time history.
- **surface faulting.** Permanent offsetting or tearing of the ground surface by differential movement across a fault in an earthquake.
- **uniform hazard response spectrum.** A plot of a ground response parameter (for example, spectral acceleration or spectral velocity) that has an equal likelihood of exceedance at different frequencies of the one degree of freedom oscillator.

## CONTRIBUTORS TO DRAFTING AND REVIEW

Ake, J.	Nuclear Regulatory Commission, United States of America
Fukushima, Y.	International Atomic Energy Agency
Godoy, A.	Consultant, Argentina
Gürpinar, A.	Consultant, Turkey
Kalinkin, I. Federation	JSC Atomenergoproekt, Russian
Kammerer, A.	Annie Kammerer Consulting, United States of America,
Nakajima, M.	Central Research Institute of Electric Power Industry, Japan
Renaud, P.	Swissnuclear, Switzerland
Serva, L.	Consultant, Italy
Sugaya, K.	Nuclear Regulation Authority, Japan
Tajima, R.	Nuclear Regulation Authority, Japan
Viallet, Emmanuele	Electricite de France, France