

**Document Preparation Profile (DPP)
Version 2 dated 04-04-2014**

1. IDENTIFICATION

Document Category	Safety Guide
Working ID:	DS487
Proposed Title:	Design of Fuel Handling and Storage Systems for Nuclear Power Plants
Proposed Action:	Revision of existing Safety Guide on <ul style="list-style-type: none"> • “Design of Fuel Handling and Storage Systems for Nuclear Power Plants”, 2003, Safety Standard Series No. NS-G-1.4
Review Committee(s) or Group:	NUSSC, WASSC, NSGC (for information)
Technical Officer(s):	K.S. Sim / NSNI

2. BACKGROUND

The purpose of the Safety Guide is to provide recommendations on how to meet the safety requirements for the design of fuel handling and storage systems in nuclear power plants (NPPs) established in Specific Safety Requirements document. The Safety Guide deals with mainly the design of fuel handling and storage systems for both unirradiated and irradiated fuels in the thermal nuclear reactors that are land based.

After the publication of the new Specific Safety Requirements for NPP Design (SSR-2/1) in 2012, a process has been established for the review and revision of the associated Safety Guides.

As part of the IAEA Action Plan¹ on Nuclear Safety, activities are ongoing to review and strengthen safety standards and improve their implementation. The revision of the Safety Guide is also proposed, under the same framework, to reflect lessons learned from the accident at the Fukushima Daiichi NPP in 2011. For example, as discussed at an international experts' meeting², design and defence in depth evaluations of spent fuel pools and associated structures, systems and components should consider events that may lead to spent fuel damage in storage (e.g., re-criticality, fuel degradation, hydrogen production and explosion, and zirconium fire during loss of cooling or loss of pool inventory). In the past, analysis of severe accidents in spent fuel pools was not widely performed. A gap assessment³ for Specific Safety Requirements in SSR-2/1 with respect to the lessons learned from the Fukushima Daiichi accident also showed a need to strengthen means for reliable monitoring of the water level and means for maintaining the cooling in the spent fuel storage system.

3. JUSTIFICATION FOR THE PRODUCTION OF THE DOCUMENT

¹ <http://www-ns.iaea.org/actionplan/default.asp>

² IAEA Report on “Reactor and Spent Fuel Safety in the Light of the Accident at the Fukushima Daiichi Nuclear Power Plant”, International Experts Meeting, 19-22 March 2012, Vienna, Austria.

³ “CSS–SECRETARIAT PROGRESS REPORT ON THE REVIEW OF THE IAEA SAFETY STANDARDS IN THE LIGHT OF THE FUKUSHIMA DAIICHI ACCIDENT”, Revision 8, 23 February 2012.

Many sections in the governing requirements document SSR-2/1, including a section for fuel handling and storage systems, are being revised to reflect lessons learned from the Fukushima Daiichi accident in 2011 under project DS462. Therefore, the revision of NS-G-1.4 should also take place in line with developing process of DS462 for revision of SSR-2/1.

Furthermore, the current NS-G-1.4 was issued in 2003 based on the previous Specific Safety Requirements document (NS-R-1, issued in 2000), while the previous requirements document NS-R-1 was superseded by the new Specific Safety Requirements document (SSR-2/1) in 2012. Hence, additional requirements related to design of fuel handling systems in SSR-2/1 need to be captured in Safety Guide NS-G-1.4.

The document for Long-Term Structure of the IAEA Safety Standards and Current Status⁴ also indicates the necessity of revising the out-dated Safety Guide.

4. OBJECTIVE AND SCOPE

This Safety Guide is intended to provide recommendations on the design of fuel handling and storage systems for NPPs. It presents recommendations on how to fulfil the requirements established in the specific safety requirements for design, SSR-2/1.

This publication is intended for use by organizations designing, manufacturing, constructing and operating fuel handling and storage systems in NPPs, as well as by regulatory bodies.

This publication is intended for application to thermal nuclear reactors that are land based. It addresses the following aspects of fuel handling and storage:

- The safe receipt of unirradiated fuel at the NPP;
- The storage and inspection of unirradiated fuel before use;
- The transfer of unirradiated fuel into the reactor;
- The removal of irradiated fuel from the reactor;
- The reinsertion of irradiated fuel when required;
- The storage, inspection and repair of the irradiated fuel in the reactor pool and its preparation for removal from the reactor pool; and
- The handling of the transport casks.

Limited consideration is given to the handling and storage of certain core components, e.g., reactivity control devices.

The physical protection of fuel or aspects associated with the safeguarding of nuclear material is also considered for information.

Aspects that are not within the scope of this safety guide include:

- In-core fuel management and associated reactor physics considerations;
- The design of fuel handling system components that are irrelevant to fuel design;

⁴

<http://www-ns.iaea.org/committees/files/CSS/205/status.pdf>

- The design of transport casks;
- Fuel storage over the long term, exceeding the design lifetime of the NPP;
- Loading of damaged fuel into transport casks with fuel exposed to coolant; and
- Design of storage facilities for spent fuel, which are not an integral part of an operating NPP, although such facilities may be located on the same site.

5. PLACE IN THE OVERALL STRUCTURE OF THE RELEVANT SERIES AND INTERFACES WITH EXISTING AND/OR PLANNED PUBLICATIONS

This Safety Guide will provide guidance on compliance with the regulatory aspects of the Safety Requirements on: Safety Assessment and Verification for Nuclear Facilities (GSR Part 4, 2009); and Safety of Nuclear Power Plants: Design (SSR-2/1, 2012).

The following IAEA documents to be interfaced with revision of NS-G-1.4:

- “Fundamental Safety Principles”, SF-1, 2006.
- “Safety of Nuclear Power Plants: Design”, SSR-2/1, 2012. (To be updated under DS462 and DS465.)
- “Safety of Nuclear Power Plants: Commissioning and Operation”, SSR-2/2, 2012. (To be updated under DS462 and DS467.)
- “Safety Assessment for Facilities and Activities”, GSR Part 4, 2009. (To be updated under DS462 and DS466.)
- “Deterministic Safety Analysis for Nuclear Power Plants”, SSG-2, 2009.
- “Storage of Spent Nuclear Fuel”, SSG-15, 2012.
- “Criticality Safety in the Handling of Fissile material”, SSG-27 (draft approved by CSS, DS407).
- “Seismic design and Qualification for Nuclear Power Plants”, NS-G-1.6, 2003.
- “Core Management and Fuel Handling for Nuclear Power Plants”, NS-G-2.5, 2002.
- “Evaluation of Seismic Safety for Existing Nuclear Installations”, NS-G-2.13, 2009.
- “Severe Accident management Programmes for Nuclear Power Plants”, NS-G-2.15, 2009. (To be updated under DS483.)

6. OVERVIEW

The table of contents (TOC) is proposed as attached.

The final contents may vary during the revision process. The majority of the text that will be reviewed and incorporated, with revisions where necessary, in the new Safety Guide is available or is being developed concurrently.

7. PRODUCTION SCHEDULE

STEP 1: Preparing a DPP	DONE
STEP 2: Approval of DPP by the Coordination	April 2014

Committee	(DONE)
STEP 3: Approval of DPP by the relevant review Committees	Q3, 2014
STEP 4: Approval of DPP by the CSS	Q1, 2015
STEP 5: Preparing the draft	Q3, 2015
STEP 6: Approval of draft by the Coordination Committee	Q4, 2015
STEP 7: Approval by the relevant review Committees for submission to Member States for comments	Q2, 2016
STEP 8: Soliciting comments by Member States	Q3, 2016
STEP 9: Addressing comments by Member States	Q1, 2017
STEP 10: Approval of the revised draft by the Coordination Committee Review in NS-SSCS	Q3, 2017
STEP 11: Approval by the relevant review Committees	Q4, 2017
STEP 12: Endorsement by the CSS	Q1, 2018
STEP 13: Establishment by the Publications Committee and/or Board of Governors (for SF and SR only))	Q2, 2018
STEP 14: Target publication date	Q3, 2018

8. RESOURCES

It is estimated to revise the Safety Guide as:

- P staff (25 man staff weeks)
- Four (4) CS meetings; 20 person weeks of non-staff
- Technical Meeting.

Attachment:

Tentative Table of Contents

The proposed TOC is made, based on TOC of the original document.

1. Introduction
 - Background
 - Objective
 - Scope
 - Structure
2. Fuel Handling and Storage Systems and Their Functions
 - General
 - Unirradiated Fuel
 - Irradiated Fuel
3. General Design Basis
 - General Considerations
 - Operational States
 - Postulated Initiating Events
 - Other Considerations
4. Systems for the Handling and Storage of Unirradiated Fuel
 - Safety Requirements
 - System Design
 - Equipment
 - Support Systems
 - Handling Operation
5. Systems for the Handling and Storage of Irradiated Fuel and Other Core Components
 - Safety Requirements
 - System Design
 - Equipment
 - Support Systems
 - Operation
 - Provision for Dismantling and Reconstitution of Irradiated Fuel
 - Provision for Damaged Fuel

- Handling and Storage of Other Irradiated Components

6. Handling of Fuel Casks

- Design for the Handling of Fuel Casks
- Equipment for Handling Fuel Casks
- Handling Operations

7. Fuel Handling at Sites with Several Reactors

8. Management System for Design

References

Annex: Flow Charts for Typical Systems for the Handling and Storage of Irradiated Fuel