DS449 - Safety Guide on "Format and Content of the Safety Analysis Report for NPPs" Addressing comments by Review Committees; second review, after resolutions to MSs' comments (Deadline for comments: 16/10/2017)

## **Resolutions to comments (for meetings of November, 2017)**

## **Comments received (2nd review):**

Finland/STUK (6), China/CAEA (5), Iran/NRPD-INRA (6), Japan/NUSSC representative (7), Canada/Health (3), <u>France</u>/ASN/IRSN-NUSSC (64); Czech Rep/SUJB (11), UK/ONR (4), France-EPReSC/ASN (2)

After deadline: <u>Korea/KINS</u> (16; 1 week); <u>Pakistan/PNRA</u> (12; 2 weeks)

## **TABLE-1, Rev.2: 44 comments from 8 Members (below):** <u>changed resolutions to comments on Chapter 19</u>

[TABLE-2: 64 comments from France/ASN/IRSN-NUSSC (separate table)]

[TABLE-3: 28 comments from Korea/KINS-NUSSC and Pakistan/PNRA (separate table)]

Rev.2: 8 November 2017

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		TABLE-	1: 44 comments from 8	Member	S		
		SECTION	N 2. GENERAL CONSIDE	RATIONS			
Phase RCs	2.15	" The full impact of any modification on	In the UK, ONR does not	X			
UK-1	Line 6	the safety of the nuclear power plant should	necessarily approve all	ļ	1		
		be evaluated and where so required	modifications.		1		
		submitted to the regulatory body for			I		
		approval before being implemented."			<u> </u>		
			<b>SECTION 3</b>				
		CHAPTER 1. INTRO	DUCTION AND GENERA	L CONSII	DERATIONS		
Phase RCs	<del>3.1.3</del>	General comment to the para.:	Geography and demography		Rev.1: See resolution	Ι	
Canada-1	General	3.2.9. This section should also cover the	should considered implications		in para. 3.2.9		
	Rev.1: It	public uses of the land and water resources	for the efficient		· ·		
1	1107.1.10	in the surrounding area, and should include	implementation of protective		l		

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	applies to para. 3.2.9, see below	an assessment of any possible interaction with the plant <u>including implications for</u> <u>off-site protective actions in an emergency.</u> "	measures under various emergency scenarios				
		СНАРТ	TER 2. SITE CHARACTER	RISTICS			
Phase RCs China-1	3.2.4 Line 1-2	This chapter of the safety analysis report should provide information concerning the site evaluation as support for the design phase, design assessment phase and periodic safety review, and comparison of the related information between the current phase and the last phase should be carried out to give a periodic conclusion of the changing trend of site condition.	The requirements of data comparison between different phases should be clarified. And the information depth or analysis differences for different phases(like PSAR and FSAR) should also be clarified.		This para will be modified as follows: " and periodic safety review, including considerations on potential changes of relevant site parameters expected over the time of the plant. This information"		
Phase RCs Japan-1	3.2.7.	3.2.7. This section should specify the site location, including both the area under the control of the <u>applicant / licensee</u> and the surrounding area in which	Clarification. The organization that submits safety analysis report is called "applicant" until a license would be issued.		" both the area under the control of the operating organization_licensee and the surrounding area in which"		
Canada-1	3.1.3 General Rev.1: It applies to para. 3.2.9, not to 3.1.3	General comment to the para.: "This section should also cover the public uses of the land and water resources in the surrounding area, and should include an assessment of any possible interaction with the plant <u>including implications for off-site</u> <u>protective actions in an emergency."</u>	Geography and demography should considered implications for the efficient implementation of protective measures under various emergency scenarios	X	The para. 3.2.9 will be modified as follows: " interaction with the plant and the implications for off- site protective actions in an emergency"		
Phase RCs Finland-2	3.2.23. End para.	For both coastal and riverine flooding, reasonable combinations of hazards should be considered (e.g. tides and strong wind).	Add: The impact of climate chance should be considered as		"For both coastal and riverine flooding, reasonable		

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		The impact of climate <del>chance_change_</del> should be considered as relevant.	<i>relevant.</i> Some of the phenomena that are expected due to the climate change are not obvious and the reminding of the issue would be useful. This implies also that the historical data is not sufficient. As an example in Finland the dominating factor of the rice of sea level at the Gulf of Finland is the melting of Antarctic.		combinations of hazards <del>should be- considered</del> (e.g. tides and strong wind) and potential changes caused by climate change should be considered."		
Phase RCs Finland-1	3.2.27 Line 5	snow loads, icing and pack ice	Please add other "winter phenomena" than snow loads should be mentioned	Х			
Phase RCs Japan-2	3.2.35	3.2.35. The feasibility of emergency preparedness in terms of plant accessibility and of transport in case of any equipment <u>necessary for</u> an emergency, including a severe accident, should be discussed in this section, taking into account all reactor units and other nuclear and non-nuclear installations on the given site, as applicable. Information provided should include availability of adequate access and egress roads for evacuation of personnel, including access to the site, and nearby population- sheltering and supply networks in the vicinity of the site.	Clarification of the definition for a relation of transportation with an emergency response actions. Reason for the deletion is that activities for nearby population in the event of an emergency will be considered in off-site emergency response plan that will be developed not by operational organization, but by organization designated by national laws.	X			
Phase RCs Japan-3	3.2.36	3.2.36 The availability of local transport networks and communications networks during and after an external event and for-	To keep consistency with the description of the draft DS484 as revision of NS-R-3 (Rev. 1).	Х			

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		the implementation of a suitable emergency						
		plan the feasibility of planning to implement emergency response actions						
		should be described. It should be ensured						
		that the requirements for adequate						
		infrastructures external to the site are met.						
Phase RCs	3.2.37	3.2.37 The needs for any necessary	Better wording.	Х				
Japan-4		administrative measures should be	The relevant responsibility in					
		identified, together with the relevant responsibilities roles of bodies and response	performing administrative measures may be identified in					
		organizations other than the operating	off-site emergency plans,					
		organization.	which will be developed by the					
			organization other than the					
			operating organization, after					
			the role of each organization					
			involved is coordinated. In the stage of preparing SAR, it will					
			be enough to identify the roles					
			of each organization involved					
			in administrative measures.					
		Chapter 3: Safety objectives an	nd design rules of struc	ctures, sy	stems and compo	onents		
Phase RCs	3.3.11	"This section should describe the measures	According to IAEA Safety	X				
Iran-1	First line	taken to prevent and to mitigate nuclear or	Glossary: Radiation					
		radiation radiological accidents"	Emergency is Nuclear or					
			Radiological Emergency.					
			Radiation is general word for					
			"nuclear or radiological".		771			
Phase RCs Finland-3	3.3.53.	3.3.53. This section should describe the protection against internal missiles. The	Please clarify: The protection against internal		<i>This para. will be modified as follows:</i>			
r manu ə		design requirements, the loads and their	missiles is normally made					
		implications, off-site protective actions and	using passive structures.		" The design			
		the required human interactions should be			requirements, the loads and their			
		specified and described with the	What is the intention of these		implications, off site			
					implications, on site			

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		justification of the successful protection. This includes the identification of all potential missile generating events, the parameters of generated missiles, including turbine missiles and any other missiles either inside or outside the containment. The design measures for ensuring the required safety level and compliance with the requirements should be presented in Chapters 4-12.	off-site actions requested?		protective actions and the required human interactions should be specified and described with the justification of the successful protection. This includes"		
Phase RCs Finland-4	3.3.54.	This section should describe the protection against high energy line breaks. The design requirements, the loads and their implications, off-site protective actions and the required human interactions should be specified and described with the justification of the successful protection. This includes the identification of all postulated failures of high energy pipelines and the dynamic effects of the pipe break and the SSCs potentially affected. The design measures for ensuring the required safety level and compliance with the requirements should be presented in Chapters 4-12.	Please clarify: The protection against High energy line breaks is normally made using passive structures. What is the intention of these off-site actions requested?		This para. will be modified as follows: " The design requirements, the loads and their implications <u>, off- site protective</u> <u>actions</u> -and the required human interactions should be specified"		
		CHA	APTER 8. ELECTRIC PO	WER			
Phase 3.4 RCs Japan-5	8.13. (c)	(c) In accident conditions with a subsequent loss of off-site power, the required safety loads can be sequenced onto the standby AC power source in case of design basis accidents without	Duplication of plant state makes confusion. "In accident conditions with a subsequent loss of off-site power" differs from "in case of		(c) In design basis accidents accident- conditions with a subsequent loss of off-site power, the		

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		overloading the primary mover and in time frames consistent with the assumptions presented in the chapter 15 on safety analyses;	design basis accidents".		required safety loads can be sequenced onto the standby AC power source, in case- of design basis- accidents without overloading the		
					primary mover and in		
Phase RCs Japan-6	3.8.13. (d)	(d) Uninterruptible AC power is continuously provided to essential safety systems and important to safety instrumentation and control systems <del>while</del> normal off site AC power systems are available and during postulated loss of off site power events <u>irrespectively the</u> availability of off-site AC power;	Better wording for clarification of the condition of off-site AC power systems. Uninterruptible AC power must be secured at any time.		time frames" (d) Uninterruptible AC power is continuously provided to essential safety systems and instrumentation and control systems important to safety, irrespectively the availability of off-site AC power instrumentation and control systems while normal off site AC- power systems are available and during- postulated loss of off- site power events;		
		CHAPTER 11. I	RADIOACTIVE WASTE	MANAGE	MENT		
Phase RCs Czech Republ-1	3.11.2 Item 1	1. The capabilities of the plant to minimize, control, collect, handle, process and store for pretreatment, treatment, conditioning and storage of liquid, gaseous, and solid radioactive waste; and	Terminology is not in compliance with the IAEA Safety Glossary, 2007 and safety standards on RAW management. Either use general term "management of	X	This resolution has been applied also to 3.11.4, 3.11.10 (see comment Czech Rep- 1bis), 3.11.14 and 3.11.16]		

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			RAW", which covers also disposal not performed at NPPs or proposed, more precise, wording.				
Phase RCs Czech Republ-2	3.11.2 Line 6	" facility (final radioactive waste repository disposal facility) and is therefore"	Terminology is not in compliance with the IAEA Safety Glossary, 2007 and safety standards on RAW management. Disposal of RAW is always a final step in RAW management.	X			
Phase RCs Czech Republ-3	3.11.2 Line 7	" However, waste acceptance criteria for repositories,"	Terminology is not in compliance with the IAEA Safety Glossary, 2007 and safety standards on RAW management.	X	" However, any waste acceptance criteria for"		
Phase RCs Czech Republ-4	3.11.29 Line 3	" Measures should also be aimed at minimizing both the volume and the activity of the waste in such a way as to meet any specific requirements, such as waste acceptance criteria, that may be posed by the design of the waste storage and disposal facility."	Interdependencies to disposal facilities (available or planned) have to be considered as well		" These Mmeasures are required to should- also be aimed at minimize minimizing both the volume and the activity of the waste (see para. 4.8 of SSR-2/1 (Rev. 1) [3]), and should be implemented in such a way so as to meet any specific criteria requirements, such as waste acceptance criteria, that are may be posed by associated with the design of the		

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Phase RCs Czech Republ-1 bis	3.11.10	3.11.10 This section should describe the capabilities of the plant to control, collect, process, handle and store for pretreatment, treatment, conditioning and storage of liquid radioactive waste"	Terminology is not in compliance with the IAEA Safety Glossary, 2007 and safety standards on RAW management. Either use general term "management of RAW", which covers also disposal not performed at NPPs or proposed, more precise, wording.	X	See resolution to Czech Rep-1 about 3.11.2		
Phase RCs Czech Republ-5	3.11.11 Bullet 3	" taking into account the interdependences among all steps in the <del>predisposal</del> -management of radioactive waste, <del>as well as the</del> <del>impact of-including</del> -the anticipated disposal option. In "	Interdependencies cover also the disposal of RAW performed "outside" the NPP.	X	" established procedures, <u>with</u> <u>account taken</u> <u>oftaking into account</u> the interdependences among all steps in the <u>predisposal</u> - management of radioactive waste, <del>as</del> <u>well as the impact of</u> <u>including</u> the anticipated disposal option"		
Phase RCs Czech Republ-6	3.11.16 (+ the rest of the text)	temporarily store intermediate store	Storage is always a temporary or intermediate option.	X			
Phase RCs Czech Republ-7	3.11.17	<del>long term</del> storage	To avoid discussion on the length of the "long term storage" considering that storage is always limited in time, it is proposed to use the term storage without any adjective.	X	Also applied to 3.11.11 (d)		

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		CHAPTI	ER 12. RADIATION PROT	<b>TECTION</b>			
Phase	3.12.14	The meaning of this sentence is not clear:	The three general principles of		3.12.14. This section		
RCs	First line	"The principles of radiation Protections	radiation protection, which		should describe how		
Iran-2		applied in the design should be	concern justification,		Tthe principles of		
	[Corrected in	described"	optimization of protection and application of dose limits,		radiation protection are applied in the		
	Rev.1]		according to Requirement 1 of		design taking into		
			GSR Part 3, shall be applied.		account the		
					Requirement 1 from		
					GSR Part 3 [37]		
					should be described,		
					including a		
					description of the means implemented		
					to ensure that: ()"		
Phase	3.12.14	"All practicable steps are taken to prevent	In SSR-2/1, "preventing		Bullet (e) will be		
RCs	Item (e)	mitigate exposure due to accidents with	accidents" is used several		modified as follows:		
Iran-3	item (c)	radiological	times but for exposure				
		consequences, including analysis of	"mitigate" or minimize" are		(e) All practicable		
		potential accidents and response with	used (Requirement 5).		steps are taken to prevent-avoid or		
		countermeasures;"	According to paragraph 3.12.2		minimize exposures		
			of this draft: " Potential		due to accidents with		
			radiation exposures of workers		radiological		
			in the nuclear power plant		consequences,		
			under accident conditions,		including analysis of		
			including those with core melting, should be addressed		potential accidents		
			and the means and other		and response with		
			measures taken to minimize		countermeasures;"		
			the exposure described."				
		CHAPTER 14. PLAN	NT CONSTRUCTION AN	D COMMI	ISSIONING		
Phase RCs	3.14.6	" The process established for the licensee	The IAEA glossary defines		" The process		

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UK-2	Line 2	to develop and approve its own test procedures, to control test performance and to review and approve test	<ul> <li>approval as regulatory</li> <li>approval. I am assuming here</li> <li>you really mean approval by</li> <li>the licensee's management,</li> <li>hence the clarification.</li> <li>Please see text of comment</li> <li>number 1 if you really do mean</li> <li>regulatory approval as in the</li> <li>UK, ONR does not approve</li> </ul>		established for the licensee to develop and approve test procedures, to control test performance and to review and approve test		
Phase RCs UK-3	3.14.8 Bullet 3 and Bullet 5	Bullet 3:         " and approve individual commissioning procedures by the licensee, including the organizational units         Bullet 5:        and approval by the licensee of commissioning	See above comments. In the UK, ONR does not approve commissioning procedures.	Х			
		СНА	APTER 15. SAFETY ANAL	<b>AYSIS</b>			
Phase RCs Czech Republ-8	3.15.35 Line 2	" Analyses of events associated with spent fuel pools and radioactive waste processing management systems	See comment No. 1	Х			
		CHAPTER 1	8.: HUMAN FACTORS EN	NGINEER	ING		
Phase RCs UK-4	3.18.35 Bullet 2	" to the licensees approved design;"	See above comments. In the UK we do not approve designs, only activities on site.		"training conform to the <del>approved</del> design intent;"		Final/ detail design might be slightly different than the one somehow accepted by licensee/regulator
			<b>19. EMERGENCY PREP</b>				
NOTE: Th		II the Chapter 19 will be entirely revised la manual during the Meeting of EPROSC 5					
	<mark>discussio</mark>	ns held during the Meeting of EPReSC-5	(8 November 2017). <u>The comm</u>	nents/resolu	itions below will be n	o longer appli	<u>cable</u>

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Phase RCs Iran 4	3.19.1. Line 2	"reasonable manner that, in <i>case of</i> an accident emergency (or incident)"	In some cases, a nuclear security event is likely to be a nuclear or radiological emergency which is very important from safety aspects. The definition of "accident" covers "any unintended event" not nuclear security event. So it is suggested to replace accident with "emergency".		" demonstrating in a reasonable manner that, in the event of an accidenta nuclear or radiological emergency, all actions necessary " ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)		
Phase RCs China-2 (marked as nr 5)	3.19.1. Line 2	" for the protection of the workers, public, and the <del>plant</del> environment could be taken,"	The principle objective of emergency preparedness is to protect people and environment, to emphasize protection of plant could affect decision making process.	X	(See Iran-5) " for the protection of the workerspublic, the publicworkers, and the plant environment could be taken," ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)		
Phase RCs Iran-5	3.19.1. Line 2	" for the protection of the public, workers, <del>and the</del> plant and enviroment could be taken"	According to SSR-2/1		See resolution to China-2, above ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)		
Phase RCs France1-1	3.19.6. Line 5 Bullet 1	Activating the Response organization (instead of "Establishing emergency management)	Establishing of an emergency management should be done at the preparedness stage	X	"Activating the response organizations Establishing emergency- management" ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)		
Phase RCs	3.19.6.		Please delete potential.	Х	"Identifying		

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Finland-5	Bullet 3	Identifying potential interface with security measures;	The interface between emergency arrangements and security measures is obvious.		potential the interface " ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)			
Phase RCs France1-2	3.19.6. Line 9 Bullet 5	Delete "Activating the response"	To be consistent with comment n° 1 (France1-1)	X	( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)			
Phase RCs China-3 (marked as nr 6)	3.19.6. Line 13 Bullet 8	Protecting emergency workers, helpers <u>and</u> <u>public</u> ;	Public is also need to be included	X	"Protecting emergency workers, andhelpers and the public;" ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)			
Phase RCs China-4 (marked as nr 7)	3.19.6. Line 19 Last bullet	Proposing the termination of on-site emergency.	The operating organization have the duty to propose the termination of on-site emergency.		( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)	Х	Last bullet refers to the "measures" for terminating the emergency	
Phase RCs Iran-6	3.19.6. After last bullet	It is suggested to add this item to the list: "Communicating with off-site officials responsible for taking protective actions and other response actions off the site or within any emergency planning zones or emergency planning distances, under the full range of emergency conditions."	One of the items in the list is : "Notifying off-site officials" which is too specific. Communicating with off-site is under Requirement 9 of GSR Part 7: "Taking urgent protective actions and other response actions"		( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)	X	Such bullet would be too detailed for the list of items. Communicating and exchange of information might be part of another document.	
Phase RCs Canada-2	3.19.6. general	Revise the bullet points to align with the requirements of GSR part 7	The current bullet points are not fully aligned with GSR part 7 requirements		See resolutions above. ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)	X	No specific changes are proposed. GSR Part 7 addresses emergency preparedness and response for all the organizations involved. The description in the	

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Comment No.	Para/Line No.	Proposed new text	Reason	Accepte d	Accepted, but modified as follows	Rejected	Reason for modification/rejecti on SAR includes the roles of the operating organization; in this para. the arrangements and measures (responsibilities and necessary actions).		
Phase RCs China-5 (marked as nr 8)	3.19.8. Line 10 Bullet (c)	"(c) An off-site emergency facility in which response personnel will assess information gained from on-site measurements and off- site monitoring data, provide advice".	Response personnel in the off- site emergency facility will also assess information gained from off-site monitoring data.	X	( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)				
Phase RCs Canada-3	3.19.8. Item (d)	(d) Off-site monitoring systems for passing data and information to the regulatory body and off-site emergency response authorities_ if appropriate or if required by national arrangements	To facilitate timely response and protective action decisions, off-site emergency response authorities should also have access to this data and information, not just the regulatory body.		Bullet (d) will be modified as follows: (d) Off-site monitoring systems for passing providing_ data and information to the operating organization and to the regulatory body_ if as appropriate_ or if required by national arrangements. ( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)				
Phase RCs Finland-6	3.19.12 <i>Last bullet</i>	Describe how emergency arrangements, including <del>potential</del> interface with security measures, are integrated and coordinated with emergency arrangements of adjacent sites.	Please delete potential. The interface between emergency arrangements and security measures is obvious.	X	( <u>Note 8/12/2017</u> : See the note in the title of Chapter 19, above)				
		CHAPTER 21. DECO	MMISSIONING AND EN	D OF LIF	E ASPECTS				

No.     No.     nodified as follows     modified as follows       Phase RCs Crech Republ-9     3.21.5 Line 2     " The main differences between the <del>alternative approaches</del> decommissioning options should be explained"     There are three basic decommissioning options – immediate dismanting deformed dismanting deformed dismanting options should be explained"     " The main differences between the <del>alternative</del> options should be explained"       No.     " The main differences between the alternative stripproaches decommissioning options should be explained"     There are three basic decommissioning options – immediate dismanting deformed dismanting decommissioning options should be explained (e.g. in the protection of minimizing the generation of waste, and well as of the technological, economic, social and other relevant indicatorslaten differences for- personal, the public and the asystems, tools and equipment required dismanting "	RESOLUTION				
Comment No.         Para/Line No.         Proposed new text         Reason         Acceptel d         Accepted, modified as follows         Rejected         Reason           Phase RCs Czech Republ-9         5.21.5         " The main differences between the atternative approaches decommissioning options should be explained"         There are three basic decommissioning options - immediate dismantling, deferred dismantling, desconside din didiferences between the altexing distant difference					
No.     No.     nodified as follows     modified as follows       Phase RCs Czech Republ-9     3.21.5 Line 2     " The main differences between the alternative approaches decommissioning options should be explained"     There are three basic decommissioning options - immediate dismanting and, only in specific cases, entombment. No "alternatives" to these options should be explained (e.g. in terms of the protection and safety, the protection of minimizing the generation of sSCs and other systems, tools and equipment required distances     Modified as follows     " The main differences between the alternative" approaches decommissioning options should be explained (e.g. in terms of the readiological- consequences for personel, the public- and the environment, and optimization minimizing the generation of waste, and with as of the technological, economic, social and other relevant indicates states states and with as of the technological, economic, social and other relevant indicates states states and with as of the technological, economic, social and other relevant indicates states states and with as of the technological, economic, social and other relevant indicates states and with as states the systems, tools and equipment required diming"					
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3.21.9 Line 1	3.21.9. An estimate of the volume of <del>low</del> - and intermediate level radioactive waste should be provided. Special attention should be paid to the following aspects:"	Text does not reflect recent IAEA RAW classification (GSG-1) and is misleading – all classes of RAW can be generated at the decommissioning phase of the NPP lifetime.	X			
	ANNEX - TYPICAL TABL	E OF CONTENT OF A SA	FETY AN	ALYSIS REPORT		
ANNEX	5.12.5. Reactor heavy water collection (BWRPHWRs only)	Editorial	X			
	No. 3.21.9 Line 1	Para/Line No.       Proposed new text         3.21.9       3.21.9. An estimate of the volume of low- and intermediate level radioactive waste should be provided. Special attention should be paid to the following aspects:"         ANNEX       5.12.5. Reactor heavy water collection	ganization:       Page Of         Para/Line No.       Proposed new text       Reason         3.21.9       3.21.9. An estimate of the volume of low- and intermediate level radioactive waste should be provided. Special attention should be paid to the following aspects:"       Text does not reflect recent IAEA RAW classification (GSG-1) and is misleading – all classes of RAW can be generated at the decommissioning phase of the NPP lifetime.         ANNEX       5.12.5. Reactor heavy water collection       Editorial	ganization:       Page Of Date:         Para/Line No.       Proposed new text       Reason       Accepte d         3.21.9       3.21.9. An estimate of the volume of low- and intermediate level radioactive waste should be provided. Special attention should be paid to the following aspects:"       Text does not reflect recent IAEA RAW classification (GSG-1) and is misleading – all classes of RAW can be generated at the decommissioning phase of the NPP lifetime.       X         ANNEX - TYPICAL TABLE OF CONTENT OF A SAFETY AN	ganization:       Page Of Date:         Para/Line No.       Proposed new text       Reason       Accepte d       Accepte, but modified as follows         3.21.9       3.21.9. An estimate of the volume of <del>low- and intermediate level radioactive</del> waste should be provided. Special attention should be paid to the following aspects:"       Text does not reflect recent IAEA RAW classification (GSG-1) and is misleading – all classes of RAW can be generated at the decommissioning phase of the NPP lifetime.       X         ANNEX - TYPICAL TABLE OF CONTENT OF A SAFETY ANLYSIS REPORT         ANNEX       5.12.5. Reactor heavy water collection       Editorial       X	Page Of Date:         Para/Line No.       Proposed new text       Reason       Accepte d       Accepted, but modified as follows       Rejected         3.21.9       3.21.9. An estimate of the volume of <del>low- mid intermediate level radioactive waste should be provided. Special attention should be provided. Special attention should be paid to the following aspects:"       Text does not reflect recent IAEA RAW classification (GSG-1) and is misleading – all classes of RAW can be generated at the decommissioning phase of the NPP lifetime.       X       Image: Colspan="3"&gt;Image: Colspan="3" Image: Colsp</del>